



Nuclear fusion as a source of energy: Status and perspectives

Dr. Hans Meister

With special thanks to U. Fantz, F. Fleschner, M. Hölzl,
M. Maraschek, A. v. Müller, Th. Pütterich (IPP), P. Barabaschi, R. Pitts (ITER), W. Biel (FZJ)



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Outline



Introduction

Concepts for fusion on earth

Status of development – where are we, which challenges remain?

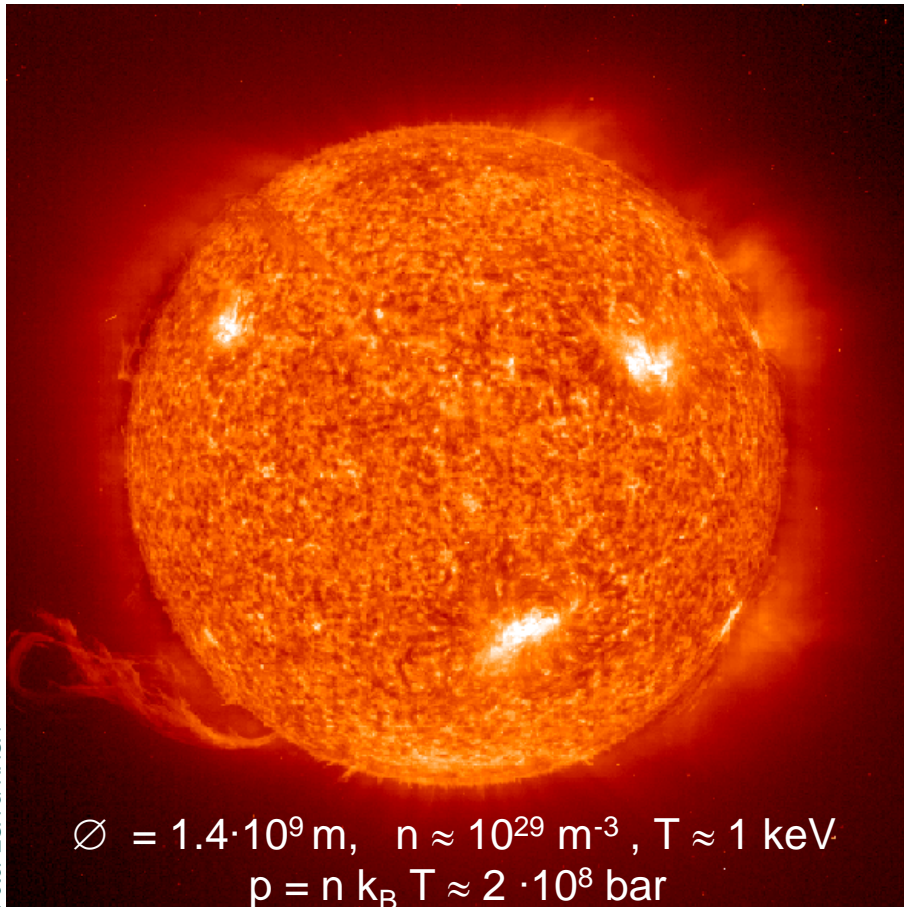
ITER – a world encompassing fusion project

Highlights from the worldwide fusion landscape

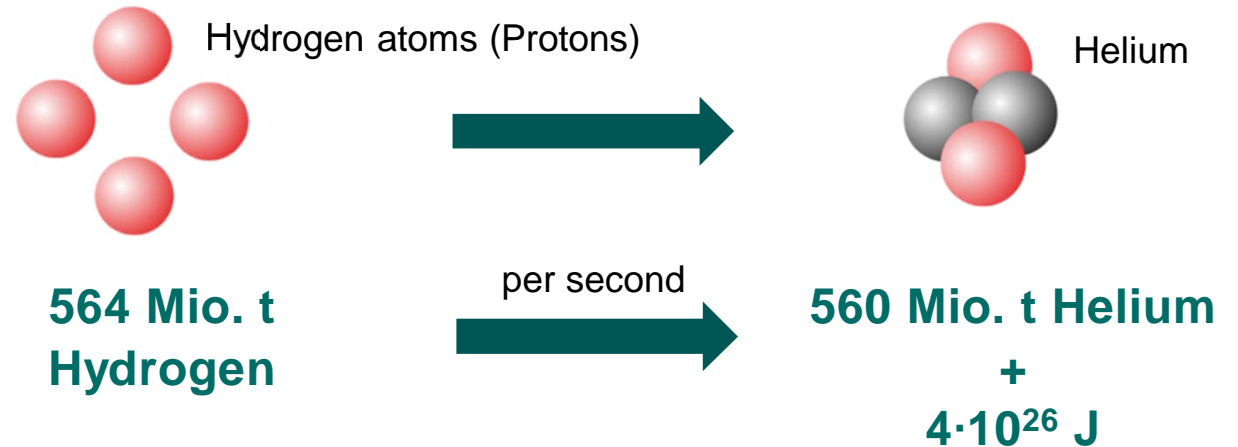
Perspectives for fusion as energy source



Nuclear fusion – the energy source of the stars



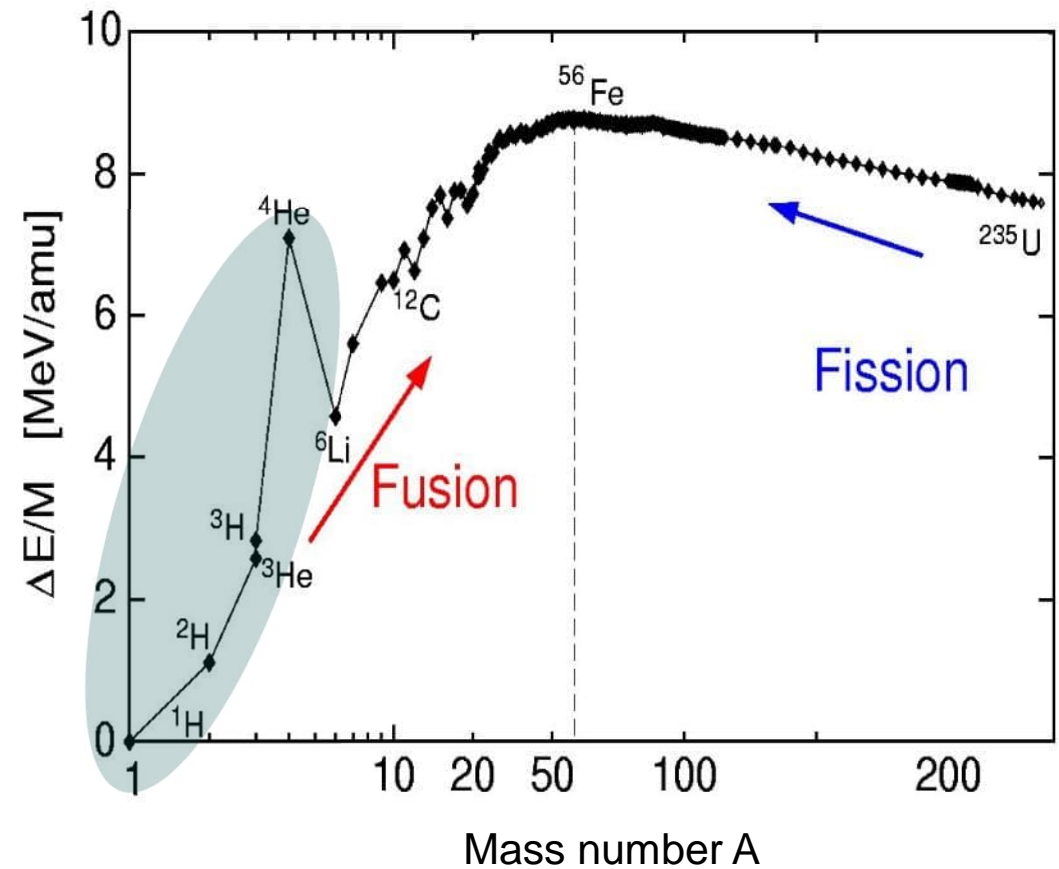
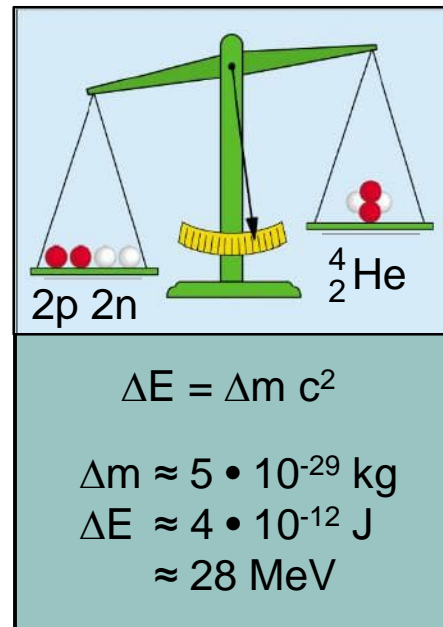
Nuclear fusion in the centre of the sun



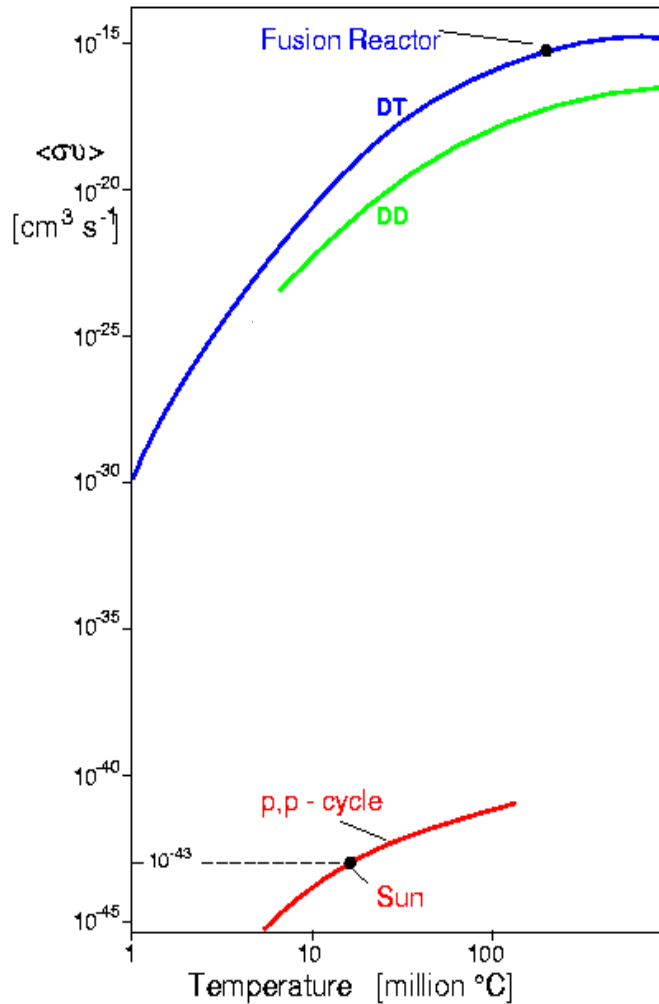
- Confinement due to gravitation
- Matter is in the plasma state

Nuclear fusion – energy production

Fusion of light atoms releases energy



Nuclear fusion – how to bring it to earth

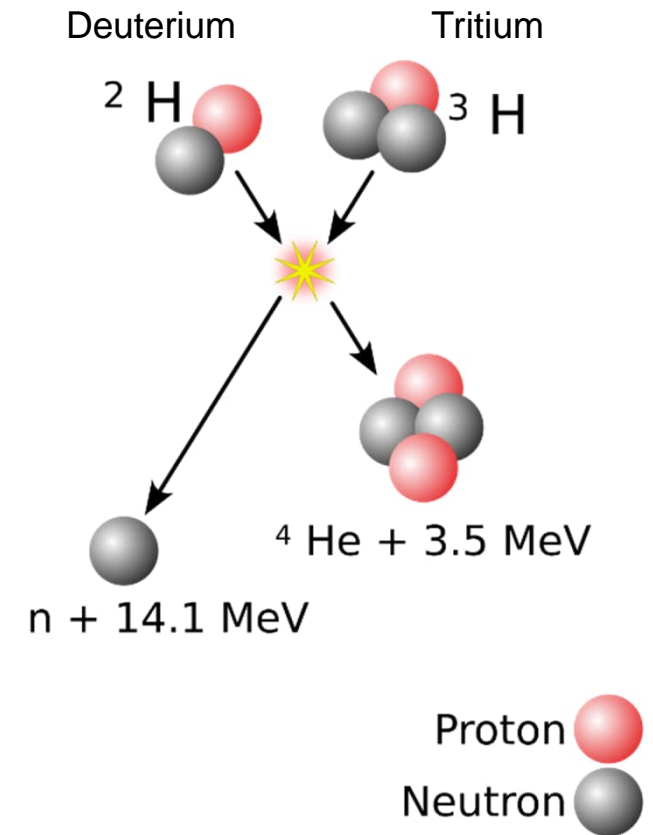


Using D and T as fuel drastically increases probability of fusion reactions

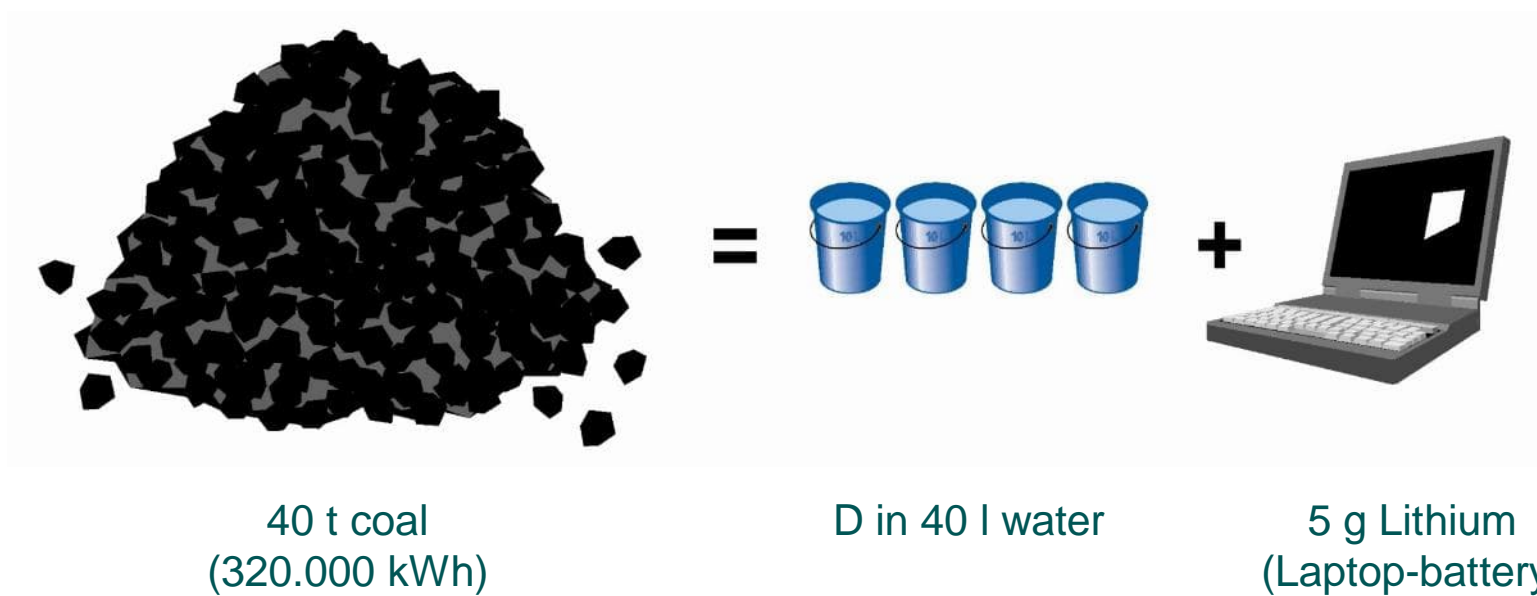
Requires temperatures above 100 Mio. $^\circ\text{C}$ ($> 10 \text{ keV}$)

Availability of fuels

- D is natural isotope of H (0.015%)
- T decays with half time of 12.3a, can be produced by



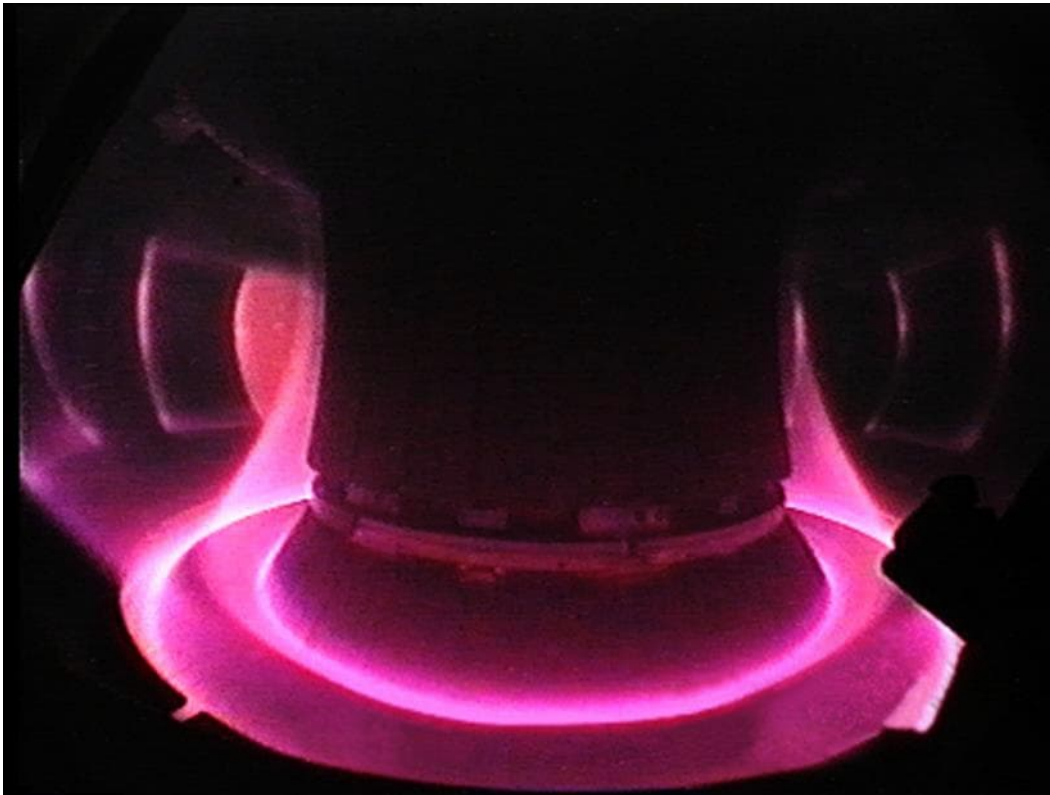
Efficiency of fuel



Fusion power plant with 1.5 GW thermal output requires about 150 kg fusion fuel per year

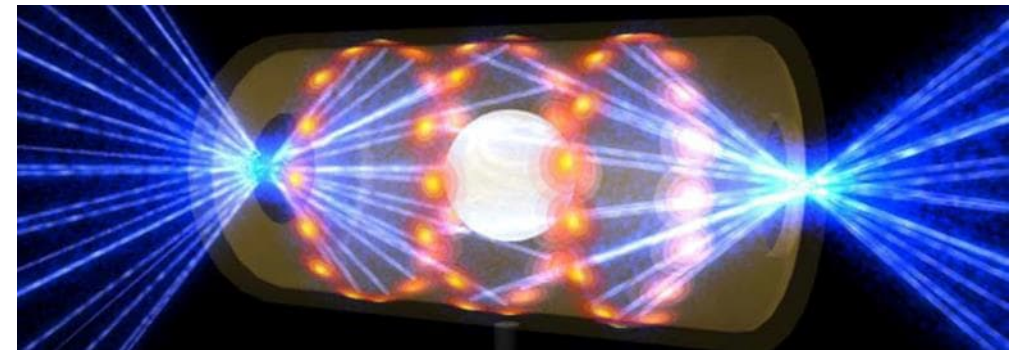
Fusion concepts – how to confine fuel on earth

Magnetically confined fusion (MCF)



Picture: MPI für Plasmaphysik

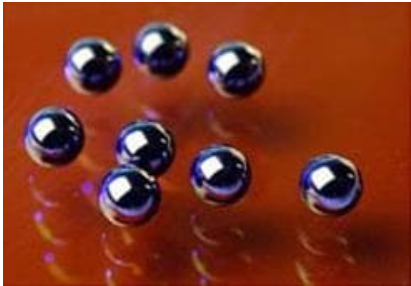
Inertially confined fusion (ICF – laser fusion)



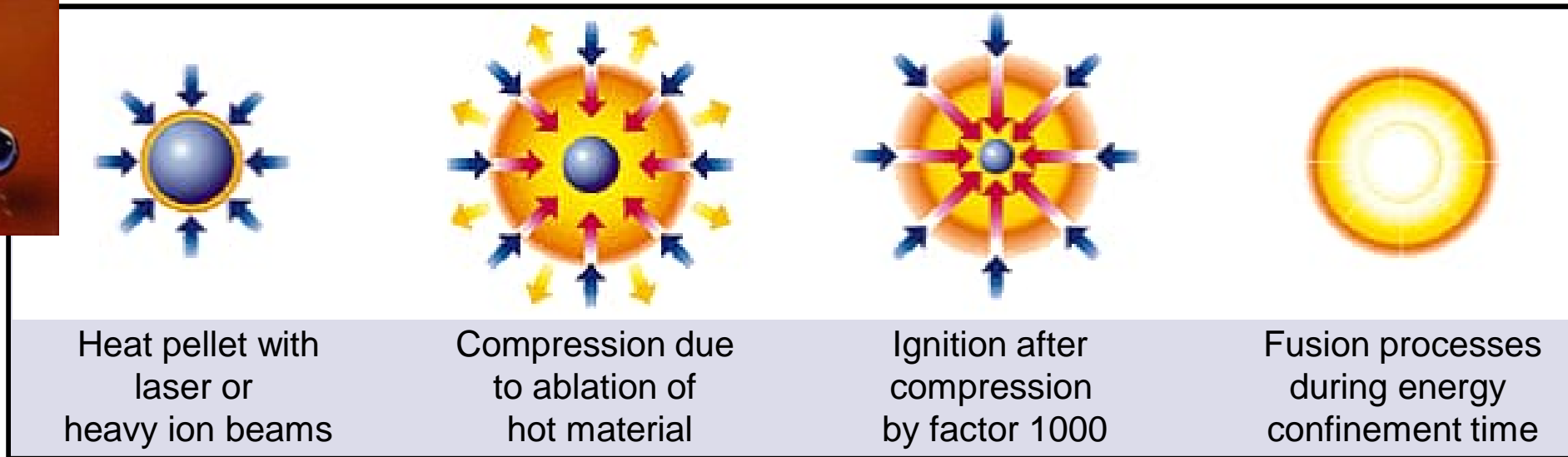
Picture: Lawrence Livermore National Laboratory/NIF

Inertially confined fusion

Small D-T-pellets are heated and compressed to initiate fusion process



Picture: Lawrence Livermore National Laboratory



Laser: 1 MJ in 10^{-10} s \rightarrow 10^{16} W

1 GJ per pellet with $r = 1$ mm
 \equiv 250 kg TNT

$T \approx 10$ keV

Sound velocity $v \approx 10^6$ m/s

$n \approx 10^{30}$ m⁻³

DT pellet with $r = 1$ mm

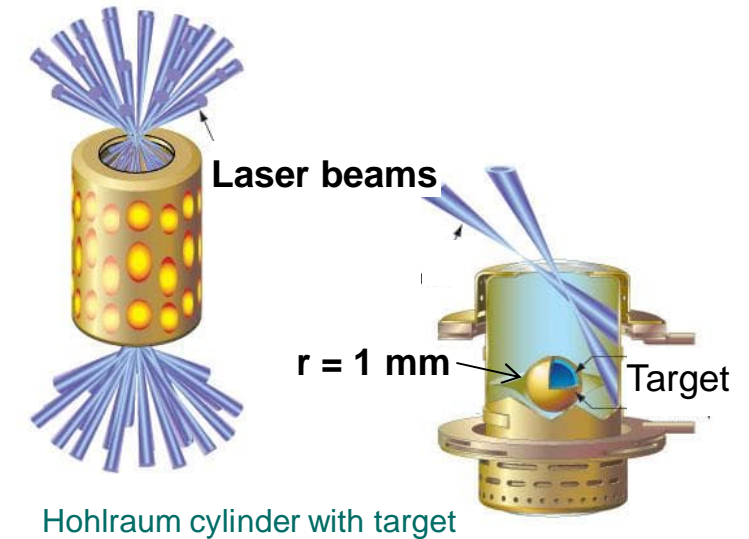
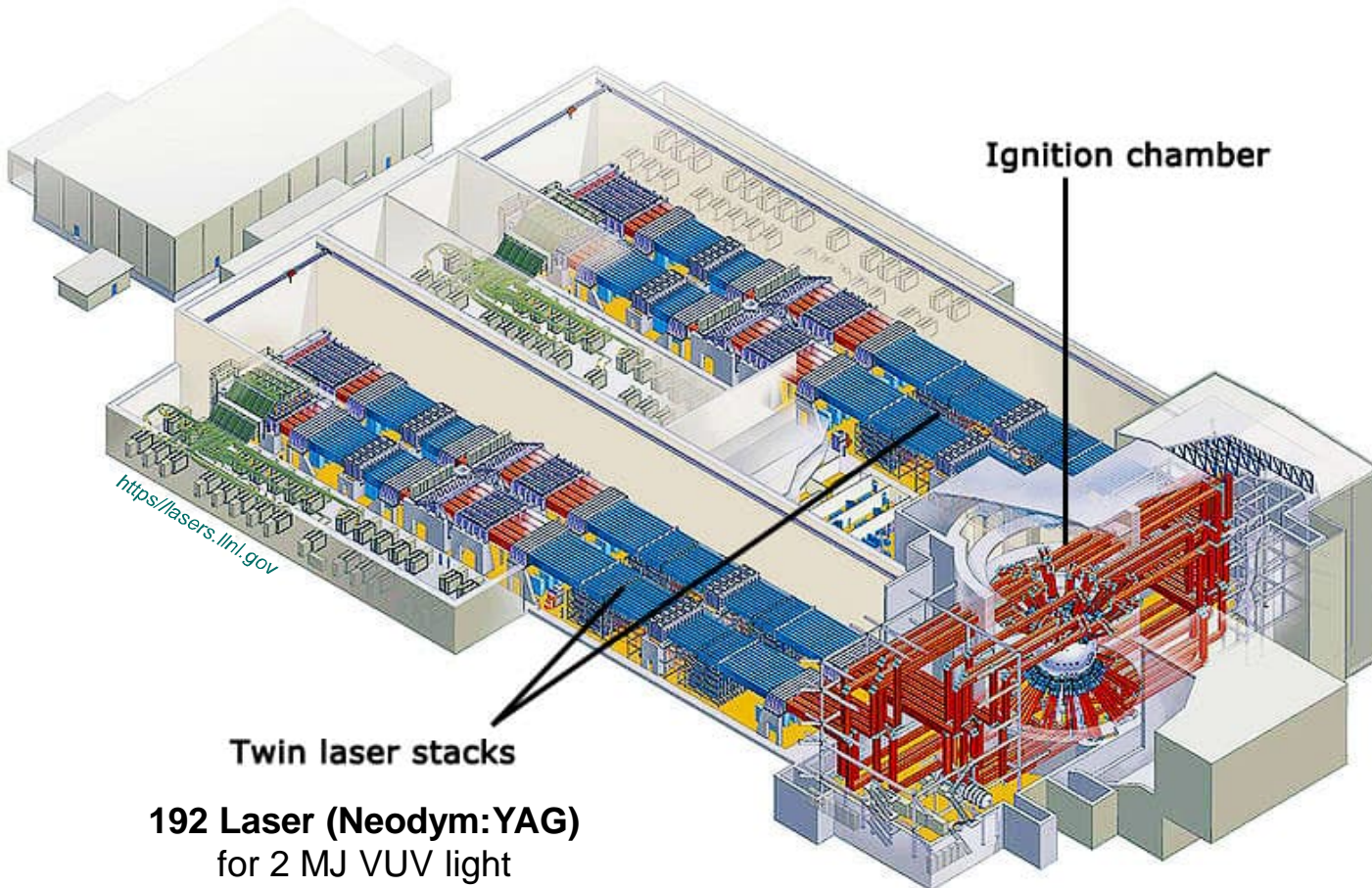
$p = n k_B T \approx 2 \cdot 10^{11}$ bar > sun

$\tau_E \approx 10^{-10}$ s = 0.1 ns

Inertially confined fusion

National Ignition Facility, NIF,
Lawrence Livermore National Lab, USA (since 2009)

- Largest laser facility worldwide



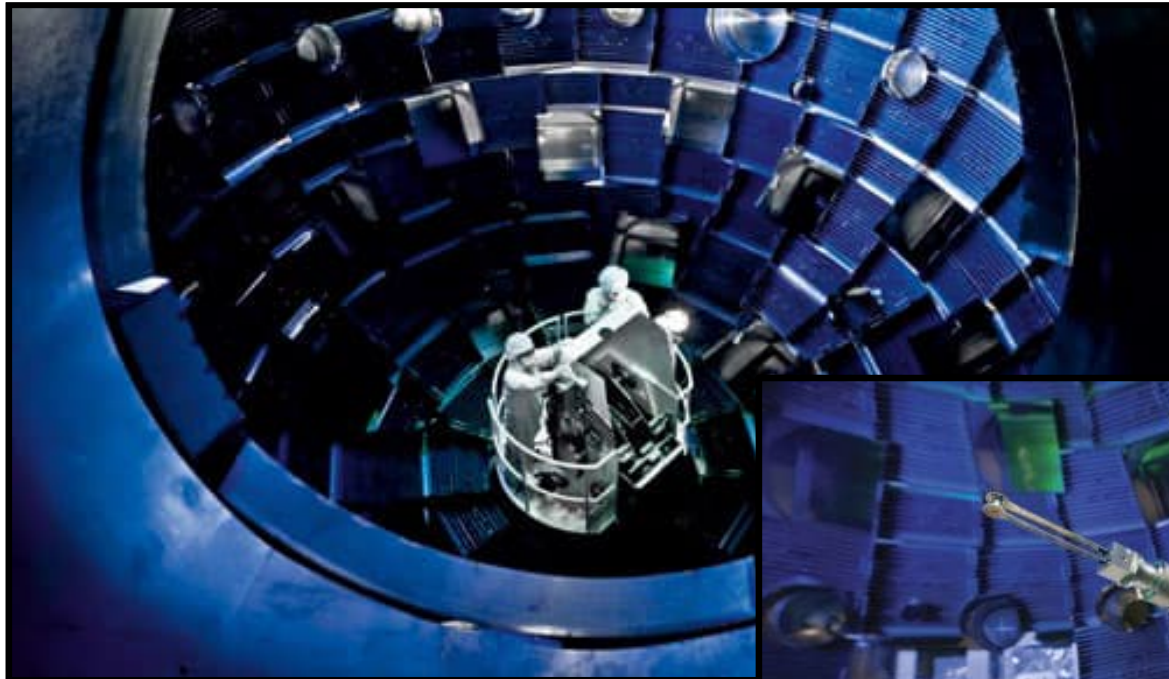
Other laser facilities

- Laser MegaJoule Project, CEA, France:
 - 2004: start construction;
 - 2022: 10 of 22 beamlines operational
- Shenguang-III (SGIII) prototype facility, China, under construction

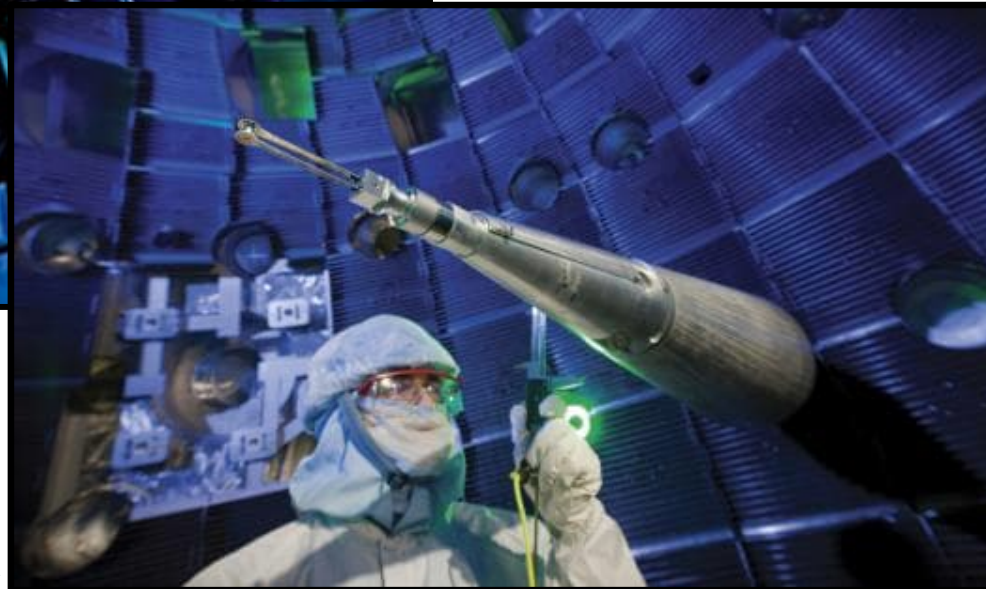


Inertially confined fusion

**Biggest success so far Dec. 2022:
3 MJ fusion power
with 2 MJ laser power on target**



NIF – View into target chamber

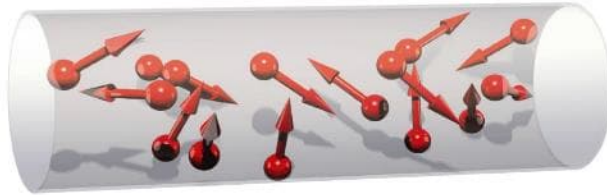


- Gain considering laser efficiency 100x lower
- **So far, 1 pellet per day. For power plant repetition at up to 10 Hz required**
- Significant technological development still needed

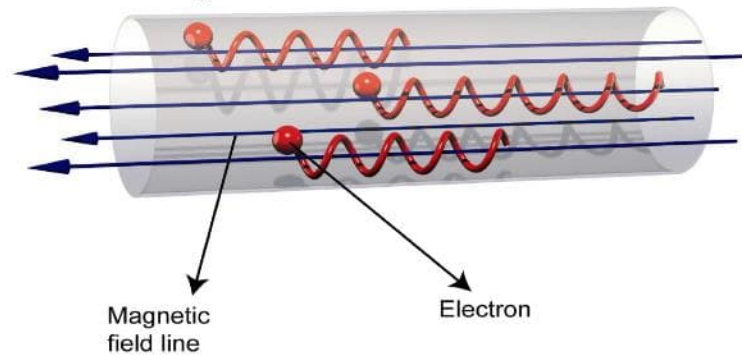
<https://lasers.llnl.gov>

Magnetically confined fusion

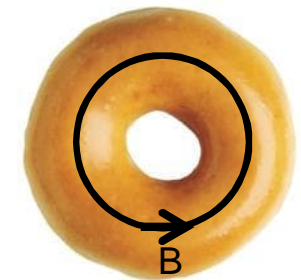
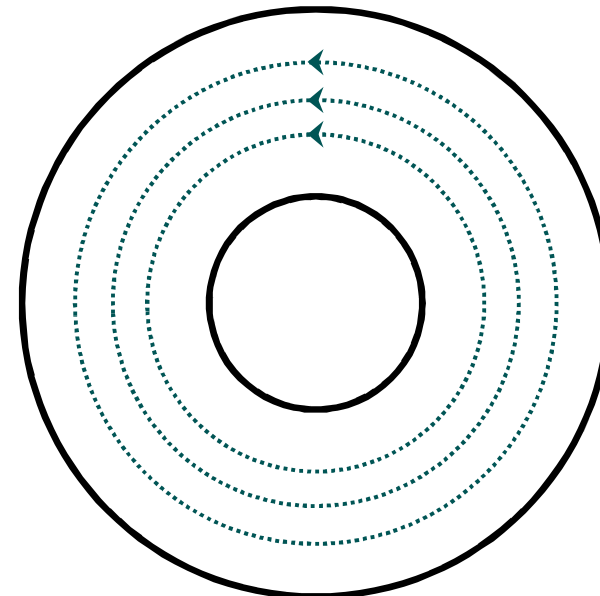
No magnetic field



With magnetic field



- Hot plasmas consist of charged particles
- Magnetic fields force them onto gyrations along field lines leading to confinement
- **Linear arrangements suffer from high losses at ends**
→ bend to torus



Larmor radius

$$r_c = \frac{m \cdot v_{\perp}}{q \cdot B} = \frac{\sqrt{2m k_B T}}{q \cdot B}$$

Example: $B = 2 \text{ T}$

$$r_{c,e} \approx 0.15 \text{ mm}$$

$$r_{c,i} \approx 6 \text{ mm}$$

Gyrofrequency

$$\omega_{c,e} = \frac{e \cdot B}{m_e}$$

$$\omega_{c,e} = 350 \text{ GHz}$$

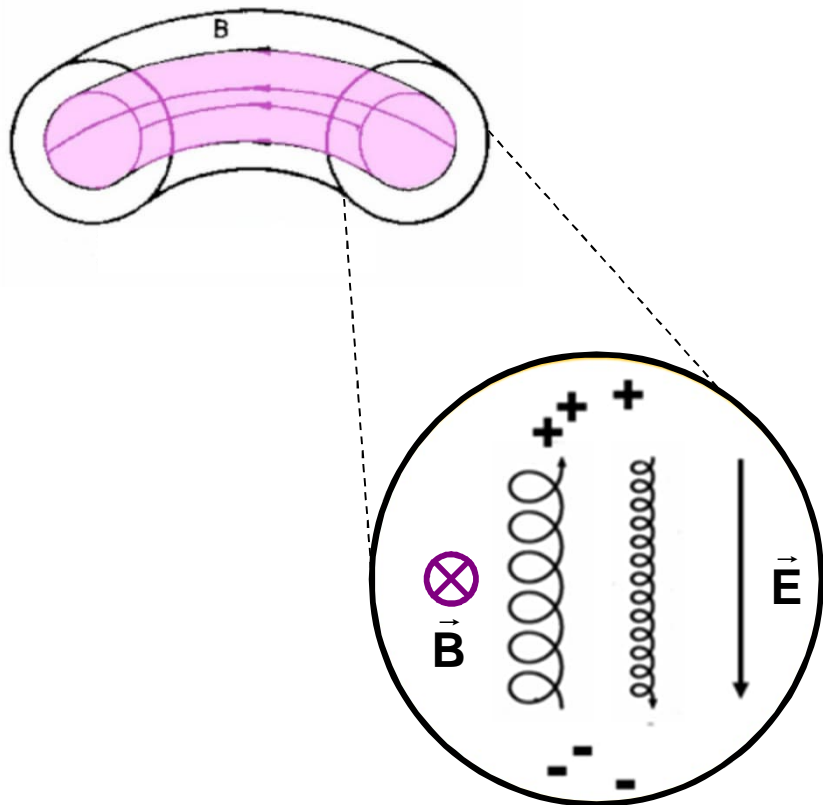
$$\omega_{c,i} = 190 \text{ MHz}$$

$T \approx 10 \text{ keV}$; $n \approx 10^{19} \text{ m}^{-3}$;
 $p = n k_B T \approx 2 \text{ bar}$
 Balanced by magnetic pressure
 $p_m = B^2 / (2\mu_0)$

$$\tau_E \approx 5 \text{ s}$$

Magnetically confined fusion

Charged particles in inhomogeneous magnetic field



- Curved magnetic field $\rightarrow \nabla B$ -force
- Centrifugal force
- Drift velocity

$$\vec{F} = -\mu \vec{\nabla} B = -\frac{mv_{\perp}^2 \vec{\nabla} B}{2B}$$

$$\vec{F} = -\frac{mv_{\parallel}^2 \vec{\nabla} B}{B}$$

$$\vec{v}_D = \frac{\vec{F} \times \vec{B}}{q \cdot B^2}$$

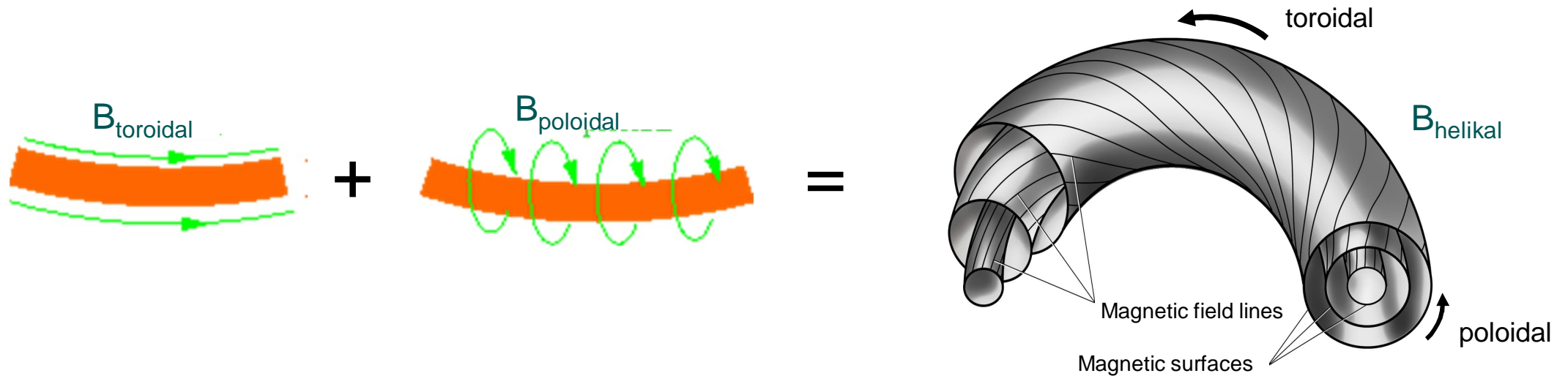
- ∇B -drift leads to charge separation
- $E \times B$ drift moves particles outwards

$$\vec{v}_D \propto \frac{E_{\text{kin}}}{q \cdot B^3} \vec{B} \times \vec{\nabla} B$$

$$\vec{v}_D = \frac{\vec{E} \times \vec{B}}{B^2}$$

Magnetically confined fusion

Purely toroidal field features drifts and low confinement → use torus with helical field lines



Transport parallel to B-field

Long mean free path of particles (≈ 20 km)

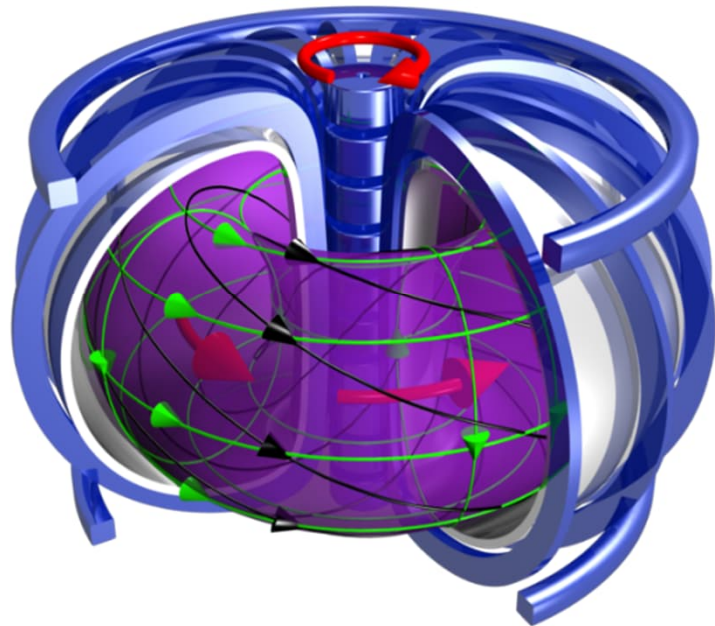
Transport perpendicular to B-field

Classical diffusion due to collisions

Magnetically confined fusion

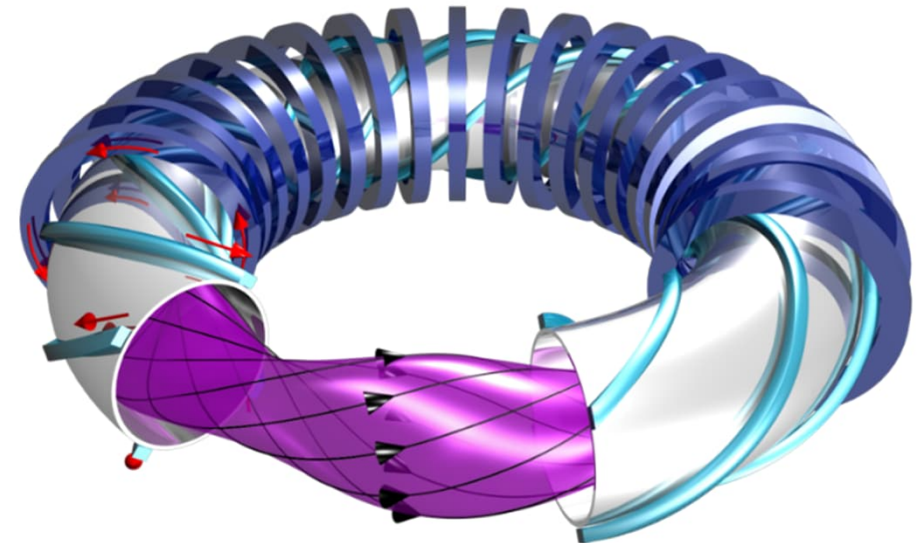
Concepts to generate magnetic field configurations

TOKAMAK
(1952 Sacharov and Tamm)



Plasma current generated via transformer

Stellarator
(1951 Spitzer)

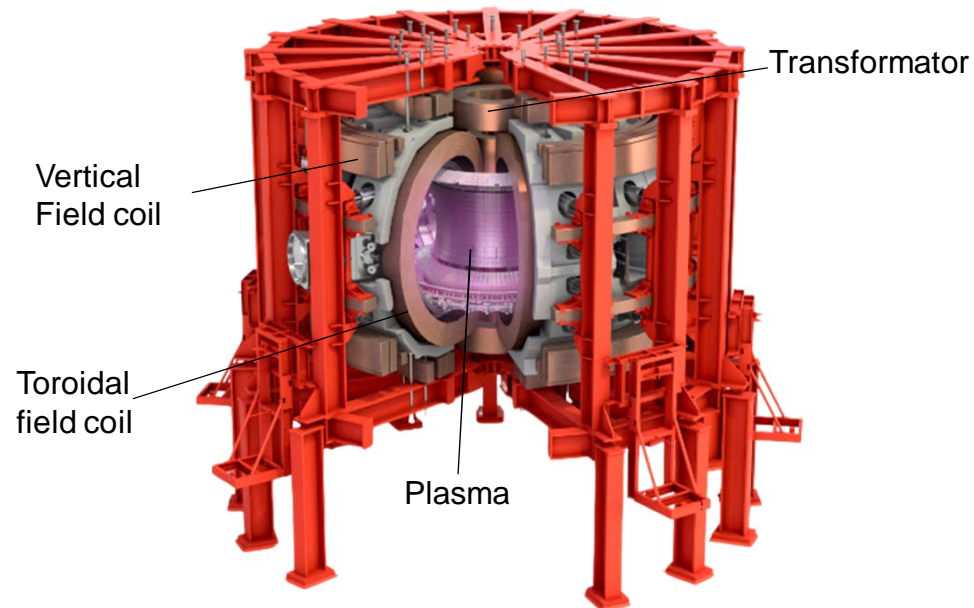


Only external field coils

Magnetically confined fusion

Tokamak: ASDEX Upgrade (Garching)

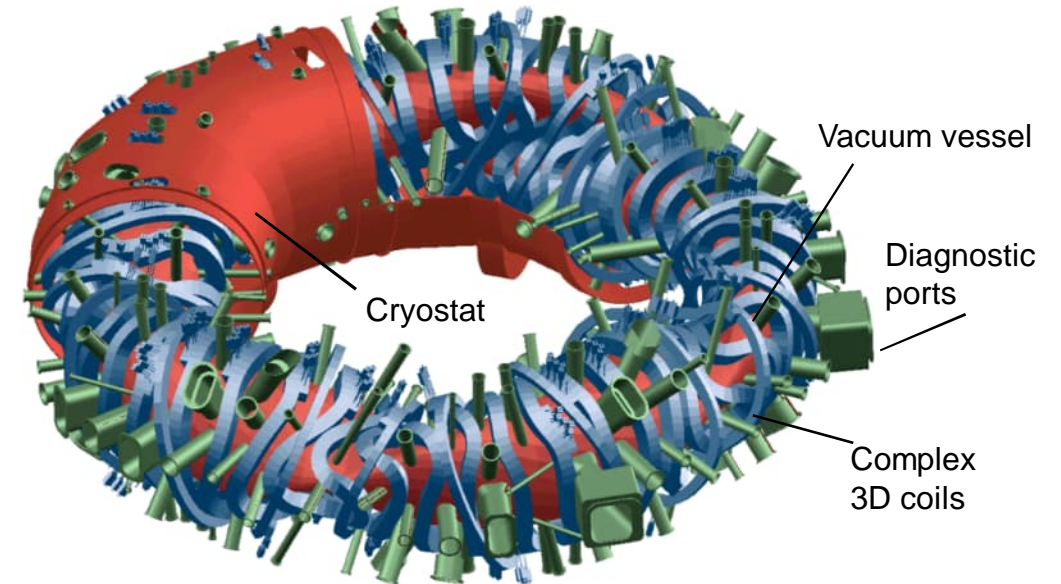
Operational since 1991
Ø 10 m, height 9 m



- 10 s plasma
- 14 m³ plasma volume, 3 mg Gas
- B = 3 T, plasma current 1.6 MA
- 30 MW external heating

Stellarator: Wendelstein 7-X (Greifswald)

2005 Construction, 2015 first plasma
Ø15 m, height 4 m

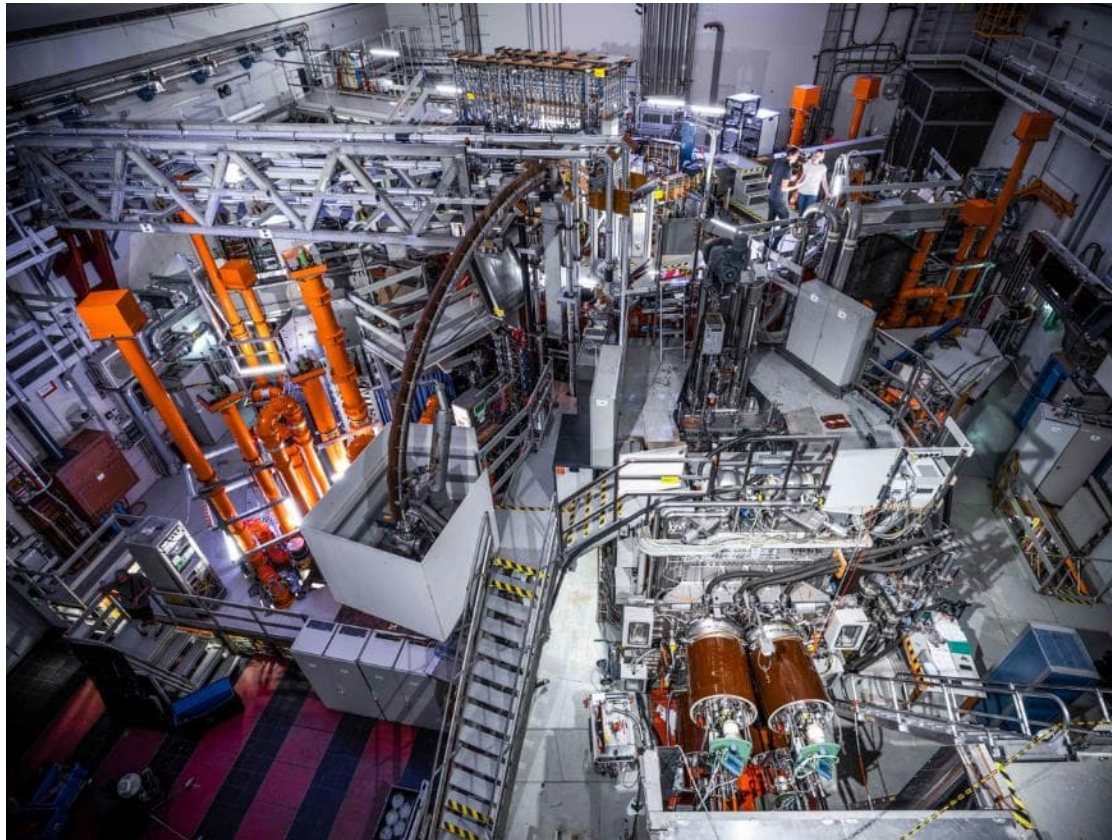


- 30 min plasma
- 30 m³ plasma volume, 5 mg gas
- B = 3 T (super conducting coils, NbTi)
- 14 MW external heating



Magnetically confined fusion

Tokamak: ASDEX Upgrade (Garching)



Stellarator: Wendelstein 7-X (Greifswald)





Magnetically confined fusion

Tokamak: ASDEX Upgrade (Garching)

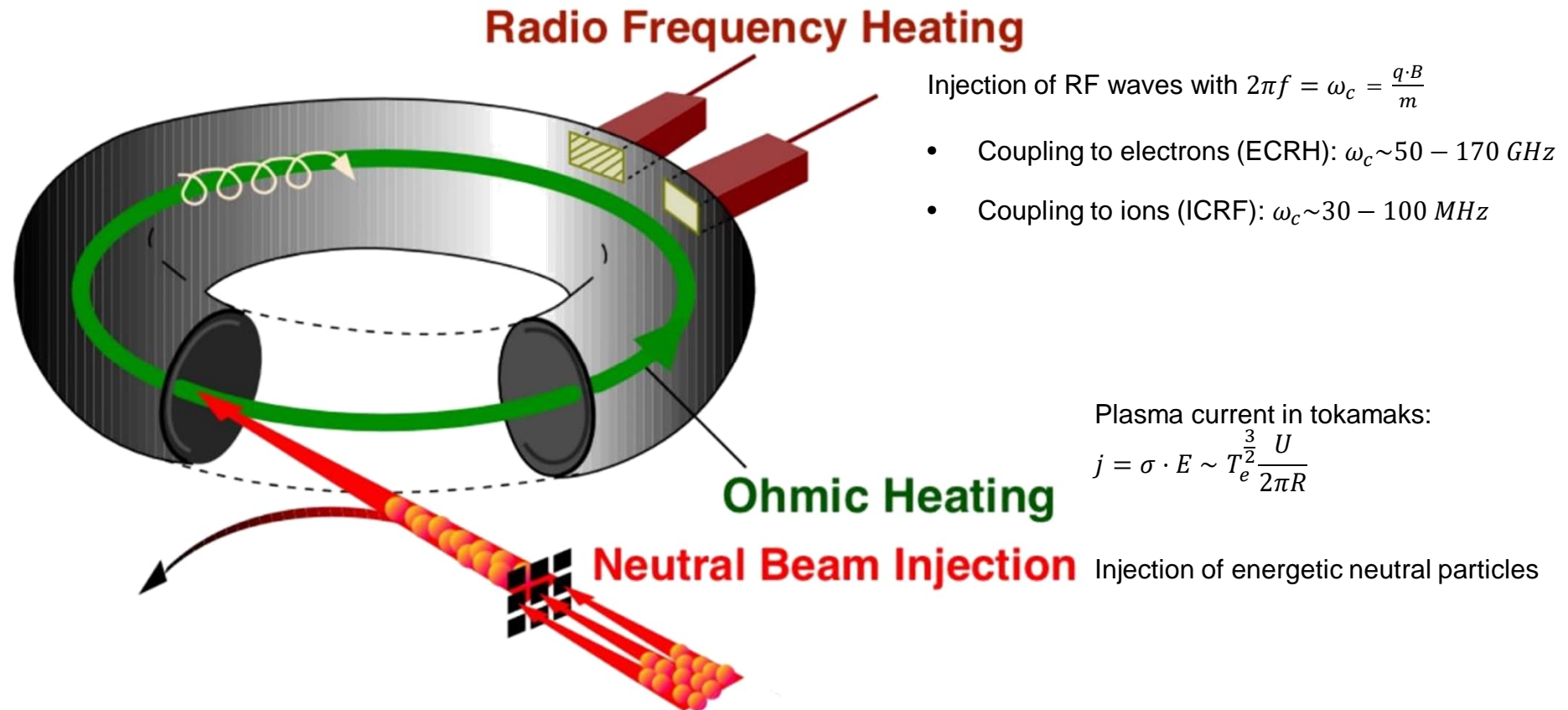


Stellarator: Wendelstein 7-X (Greifswald)



Magnetically confined fusion – External heating systems

In order to reach high plasma temperatures, various heating systems are in use

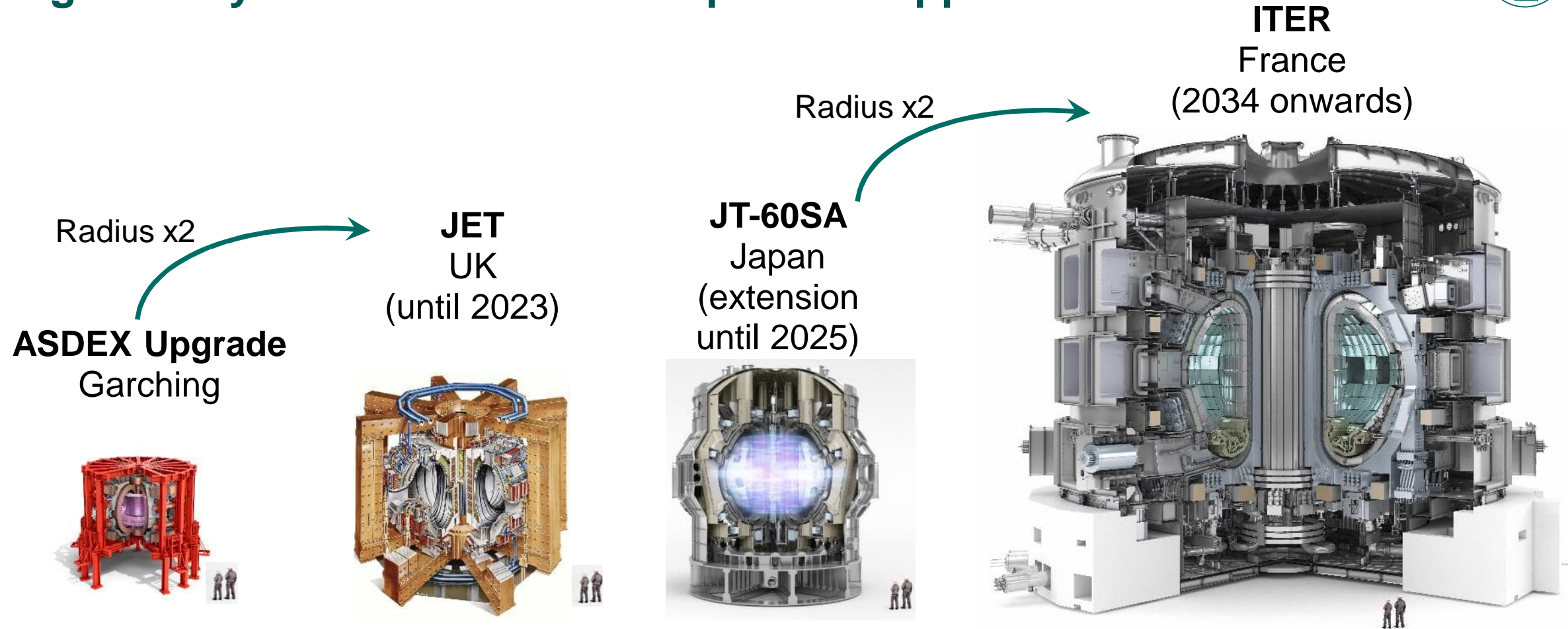


In fusion reactor:

- α -particle heating

He produced by DT-reaction remains confined and adds 3.5 MeV per particle to the plasma

Magnetically confined fusion – step ladder approach



| | | | | |
|-------------------|--------------------|---------------------|---------------------|---------------|
| 1.65 m | ~2.96 m | ~2.96 m | 6.2 m | Major Radius |
| 14 m ³ | ~80 m ³ | ~135 m ³ | ~800 m ³ | Plasma volume |



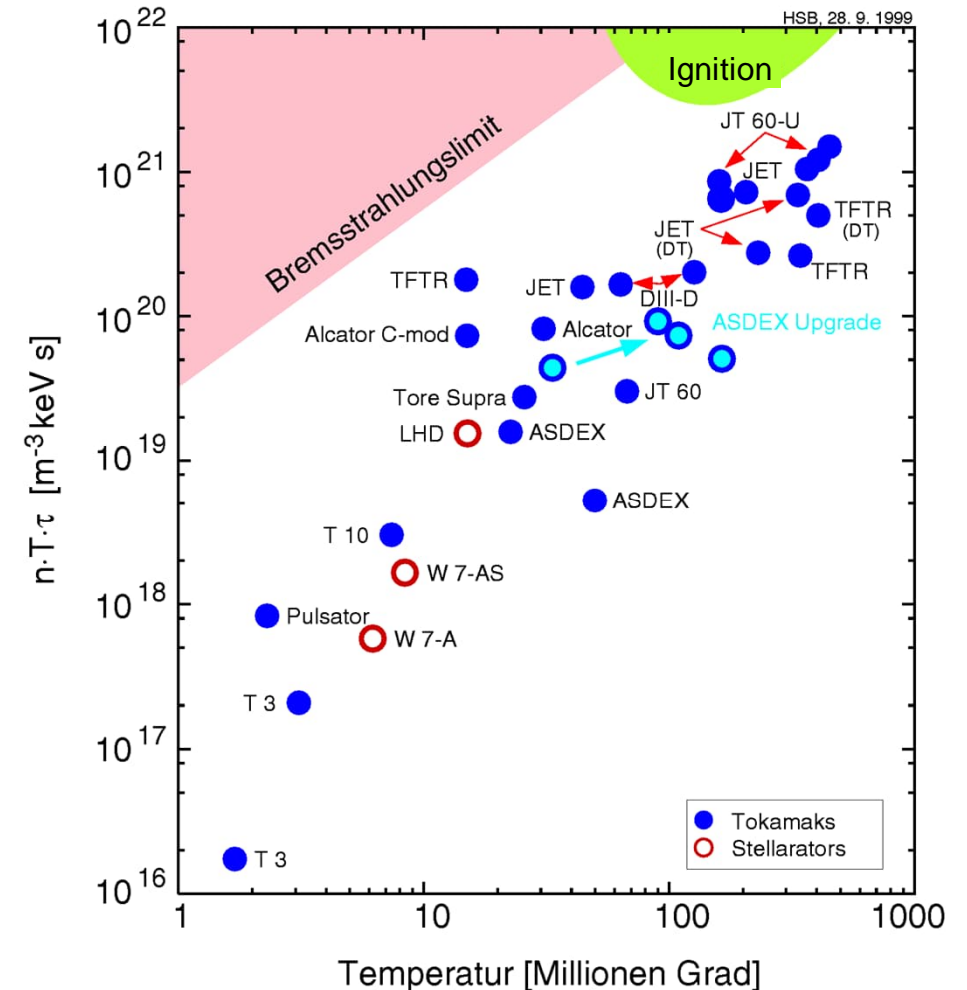
Criterion for fusion success – $n \cdot T \cdot \tau$

Requirements for burning fusion plasmas

- High temperature (T)
 - ✓ 400 Mio. °C achieved
- High particle density (n)
 - ✓ 10^{20} particles/m³ achieved
- Long energy confinement times (τ)
 - ✗ $\tau < 1$ s up to now

Improvement of energy confinement needed

➤ Larger and/or better





Recent updates on fusion achievements – world wide

- Overall progress of fusion is remarkable

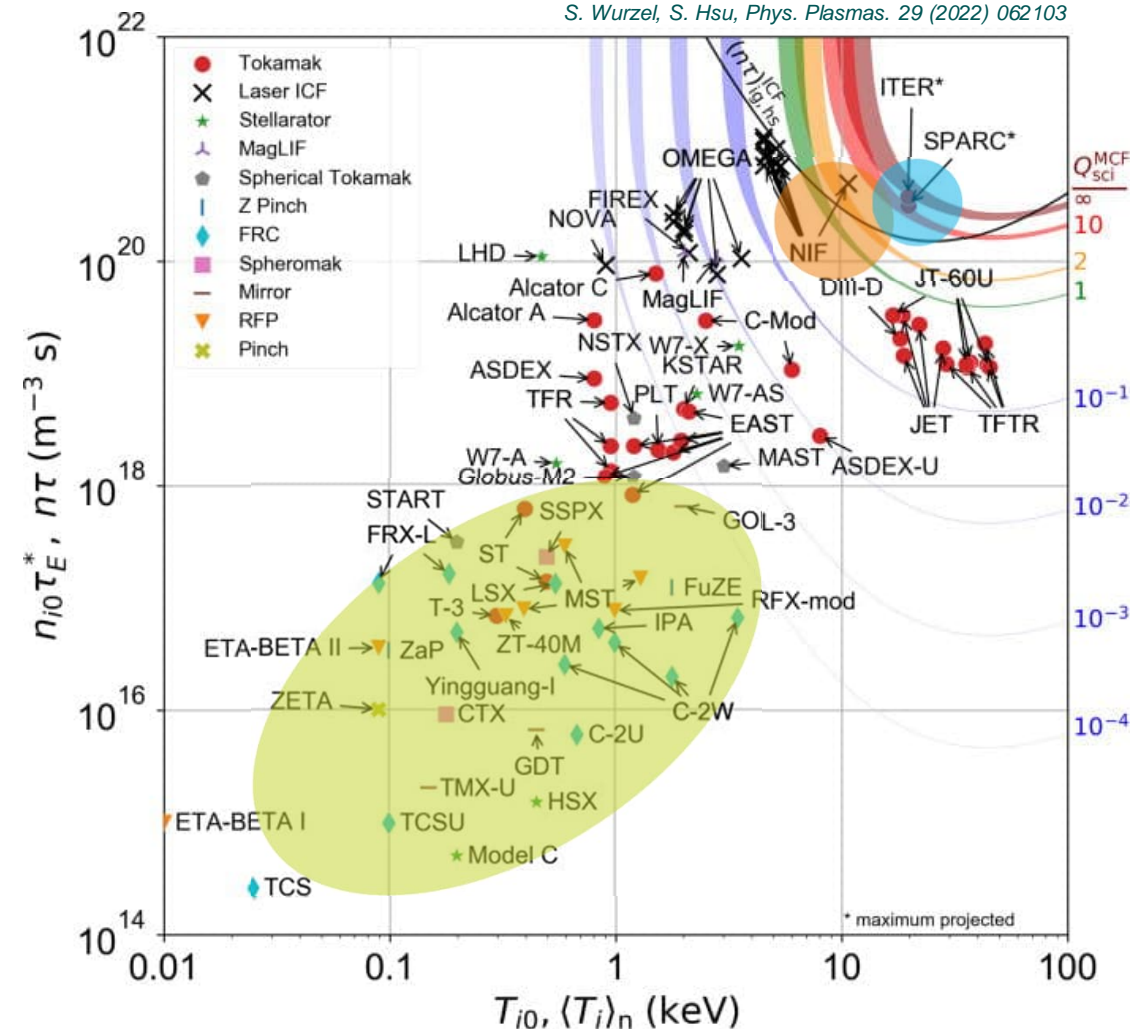
– Fusion energy gain $Q = \frac{P_{fus}}{P_{heat}} = \frac{5P_{\alpha}}{P_{heat}}$

- NIF (inertial confinement) achieved $Q=1.5$

- Field-reversed configurations (FRC) are far behind conventional magnetic confinement in terms of fusion gain

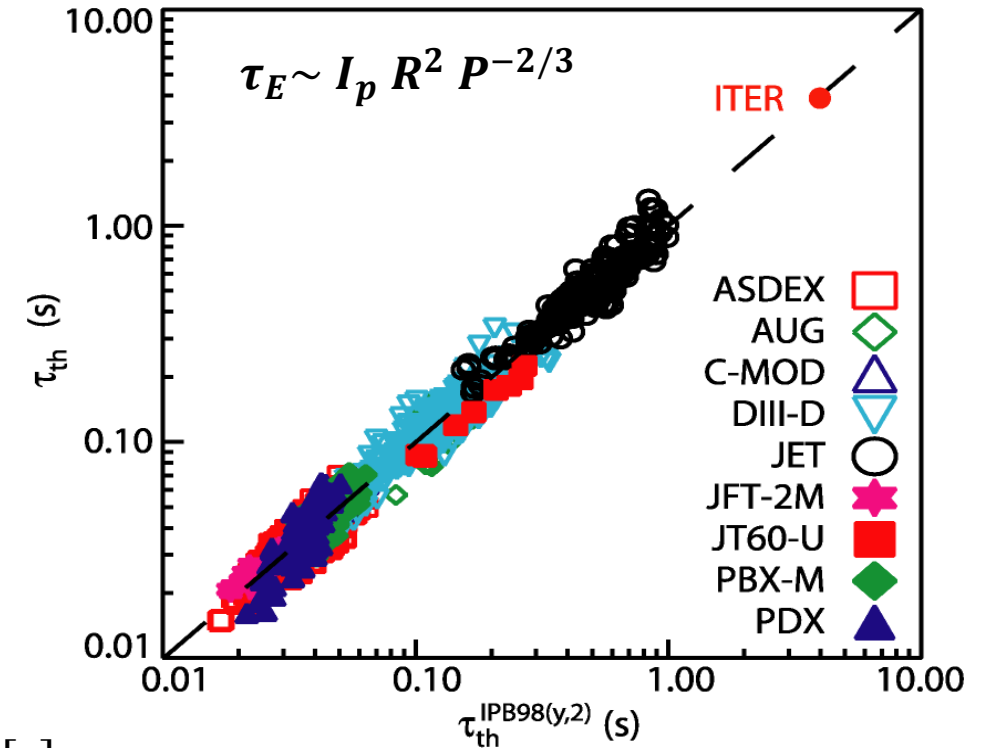
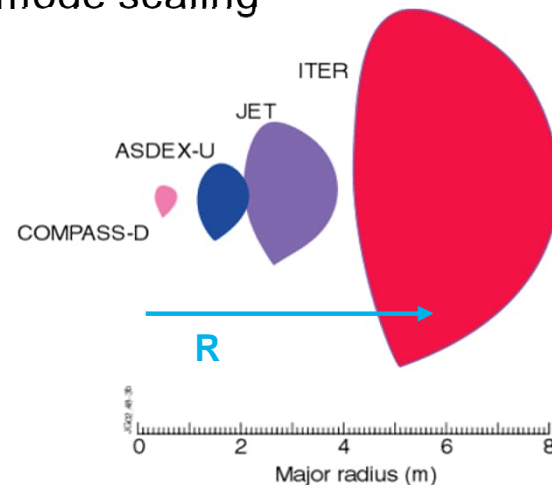
- SPARC: New private industry tokamak

- DT fuel, targets $Q=10$ for short pulses
- Start of operation 2027



Energy confinement – empirical scalings

- Energy confinement is difficult to predict quantitatively
 - use scalings from experiments + plasma physics limits
- Baseline is robust standard H-mode, achieved in every operating tokamak with divertor
- IPB98(y,2) Type-I ELMing H-mode scaling

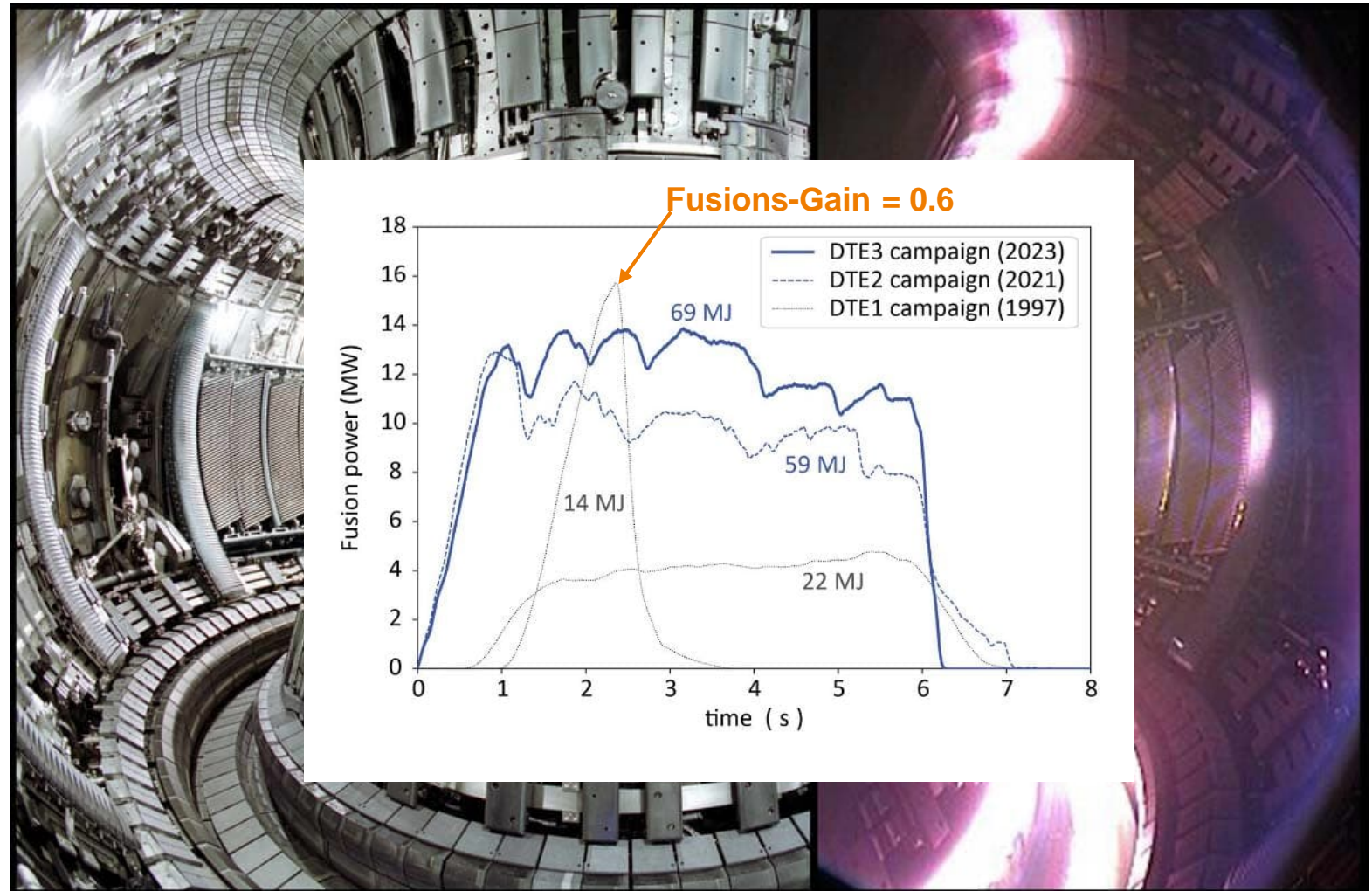


$$\tau_{E,th}^{98(y,2)} = 0.144 I^{0.93} B^{0.15} P^{-0.69} n^{0.41} M^{0.19} R^{1.97} \varepsilon^{0.58} \kappa^{0.78} \text{ [s]}$$

Recent updates on fusion achievements – JET 2023

Joint European Torus (JET)

- Largest operational facility until 2023
- 3 DT campaigns
- $Q=0.6$ over several seconds



Source: Eurofusion

Challenges in ICF

Hohlraum targets

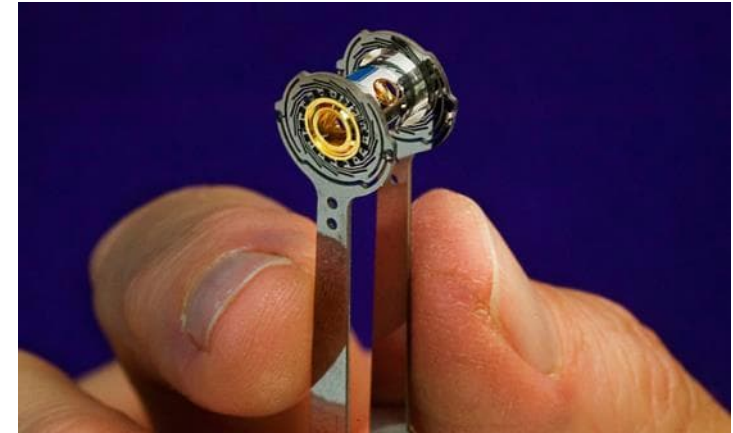
- High precision capsule with cryogenic DT pellet
- Mass production

High power lasers

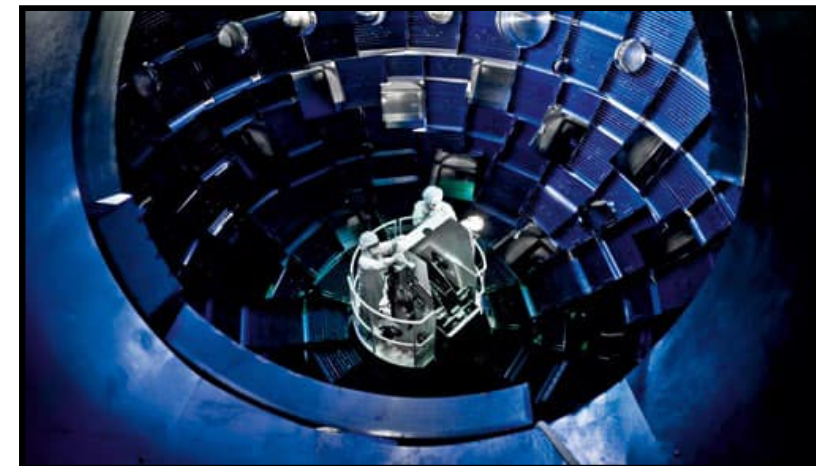
- Increase wall-plug efficiency
- Illuminate pellet uniformly to avoid instabilities

Reactor scale integration

- Reliable laser power at up to 10 Hz repetition
- Requires ignition of DT pellets at up to 10 Hz
- Alignment of pellets in laser focus
- Handling of fusion power and T breeding



Picture: LLNL



<https://lasers.llnl.gov>

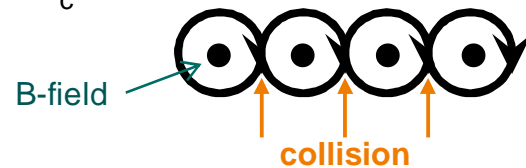
Challenges in MCF – the reality of confinement

Transport perpendicular to B-field limits energy confinement

Classical diffusion

- Displacement due to collisions by Larmor radius

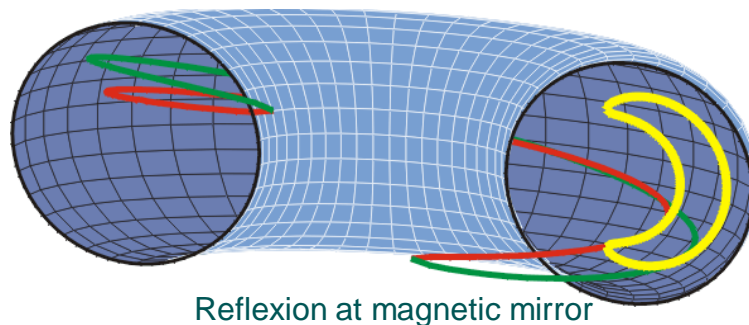
$$\Delta x = r_c$$



$$D = \frac{\Delta x^2}{\Delta t}$$

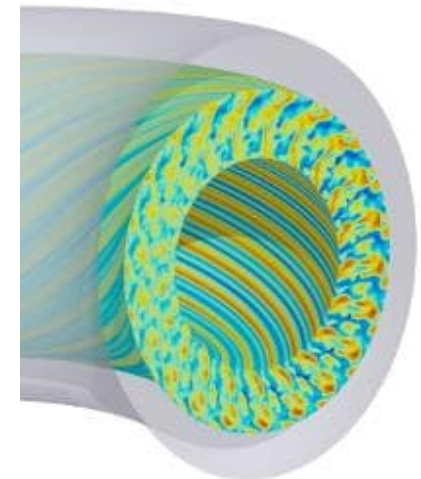
Neoclassical transport

- Collision of particles trapped on banana orbits
- Displacement is width of banana orbit



Turbulent transport

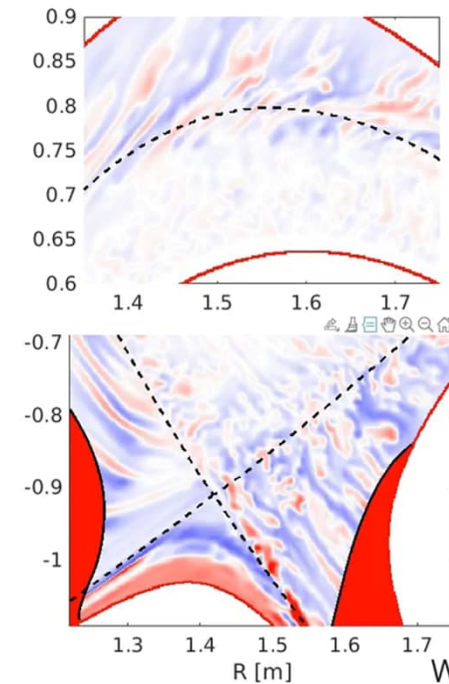
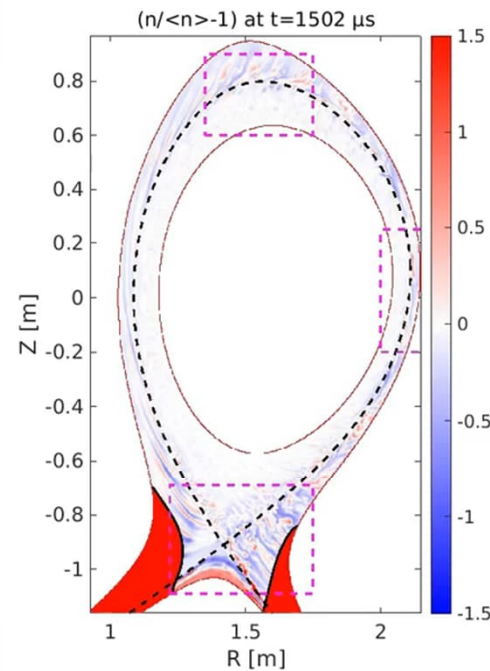
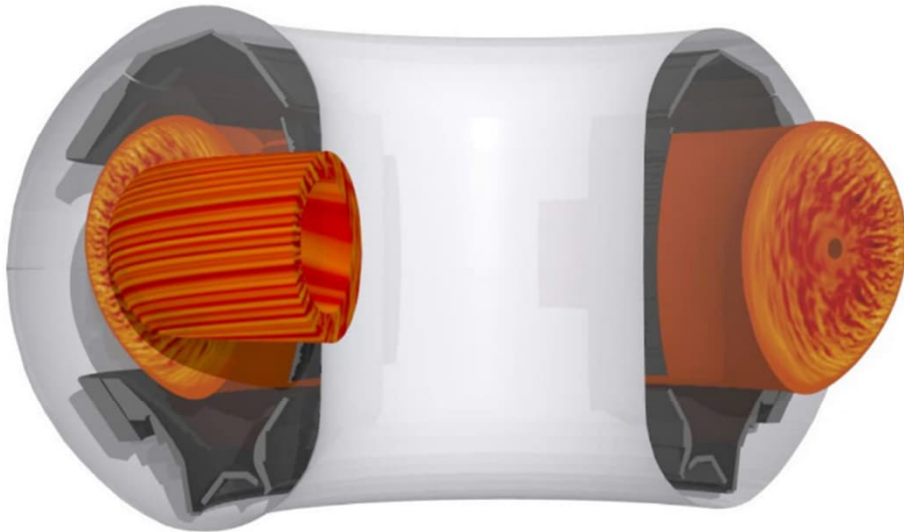
- Displacement is extent of turbulent structures



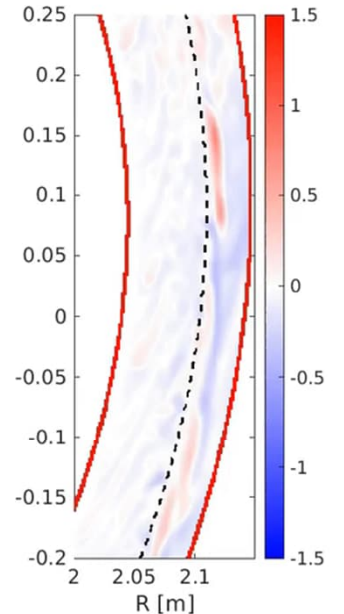
| | Δx | D | Torus |
|----------------------|-----------------|-----------------|-------|
| Classic Diffusion | r_c | D | 10 cm |
| Neoclassic Transport | $10 \cdot r_c$ | $100 \cdot D$ | 1 m |
| Turbulenz | $100 \cdot r_c$ | $10000 \cdot D$ | 10 m |

Challenges in MCF – the reality of confinement

Turbulence can meanwhile be measured and simulated



high density / collisionality



Wladimir Zholobenko, NME 2022

gene.rzg.mpg.de

GENE

- Experiments and theoretical calculations agree
- More and more phenomena can be understood ab initio

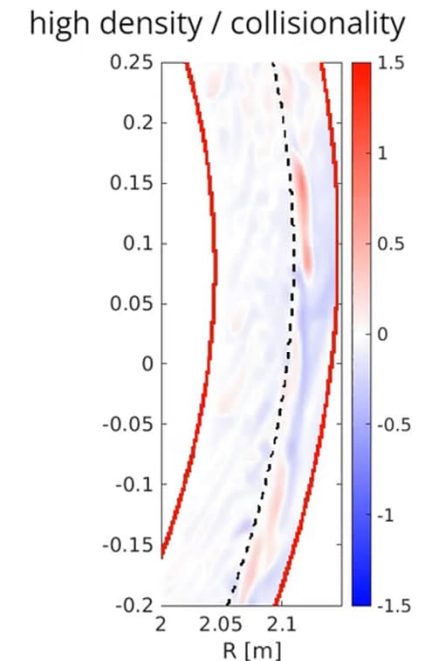
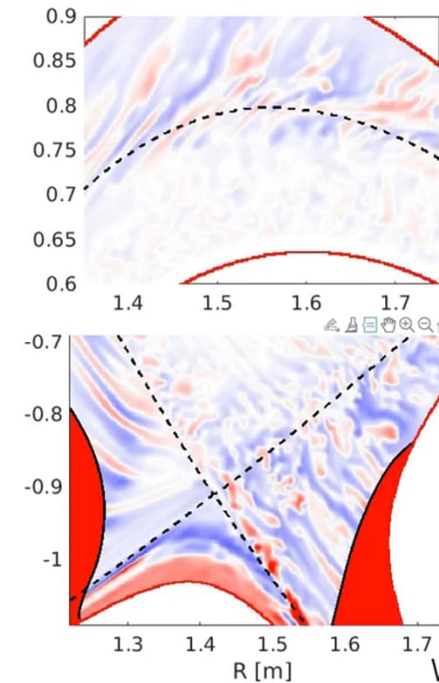
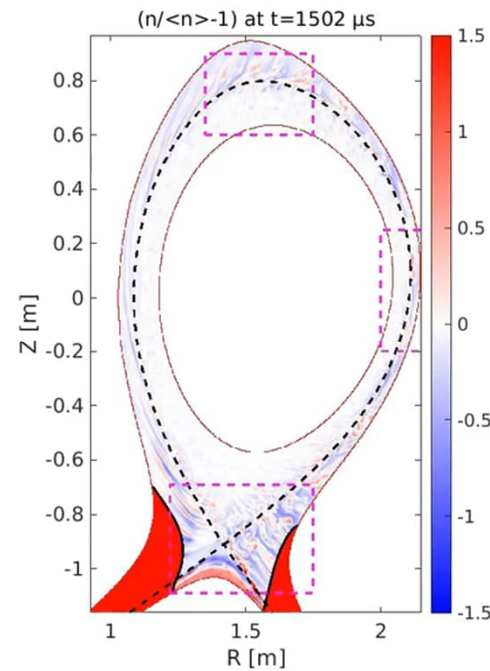
Challenges in MCF – the reality of confinement

Turbulence can meanwhile be measured and simulated



Press release, CCFE, Tuesday 26 June 2012

High-speed video image of the MAST plasma obtained at the start of an Edge Localized Mode.



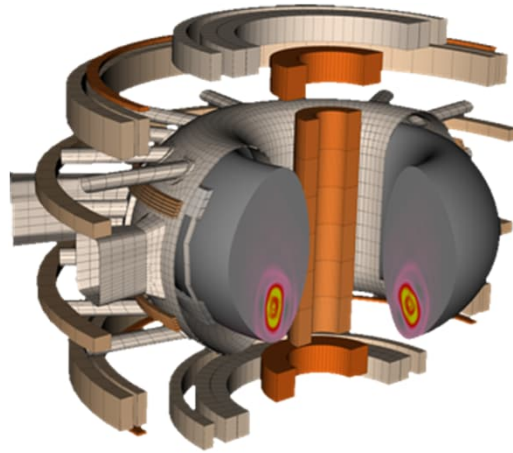
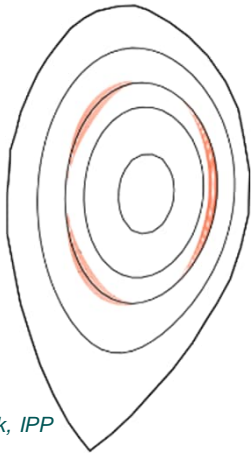
Wladimir Zholobenko, NME 2022

- Experiments and theoretical calculations agree
- More and more phenomena can be understood ab initio

Challenges in MCF – instabilities, disruptions

Plasma current as source of free energy leads to resonant magnetic flux surfaces

- Development of magnetic islands and MHD instabilities



- Can interrupt plasma current and thus confinement

➤ Disruptions



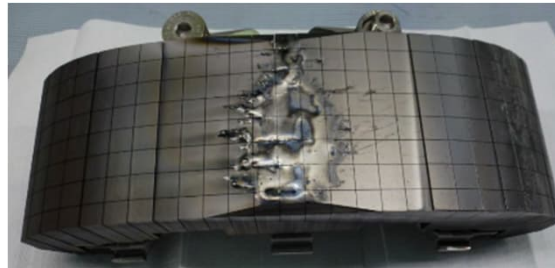
Simulation and video of a vertical displacement event in ASDEX Upgrade

Challenges in MCF – instabilities, disruptions

- **Disruptions can cause**
 - Severe mechanical and thermal loads
 - Damage to in-vessel components

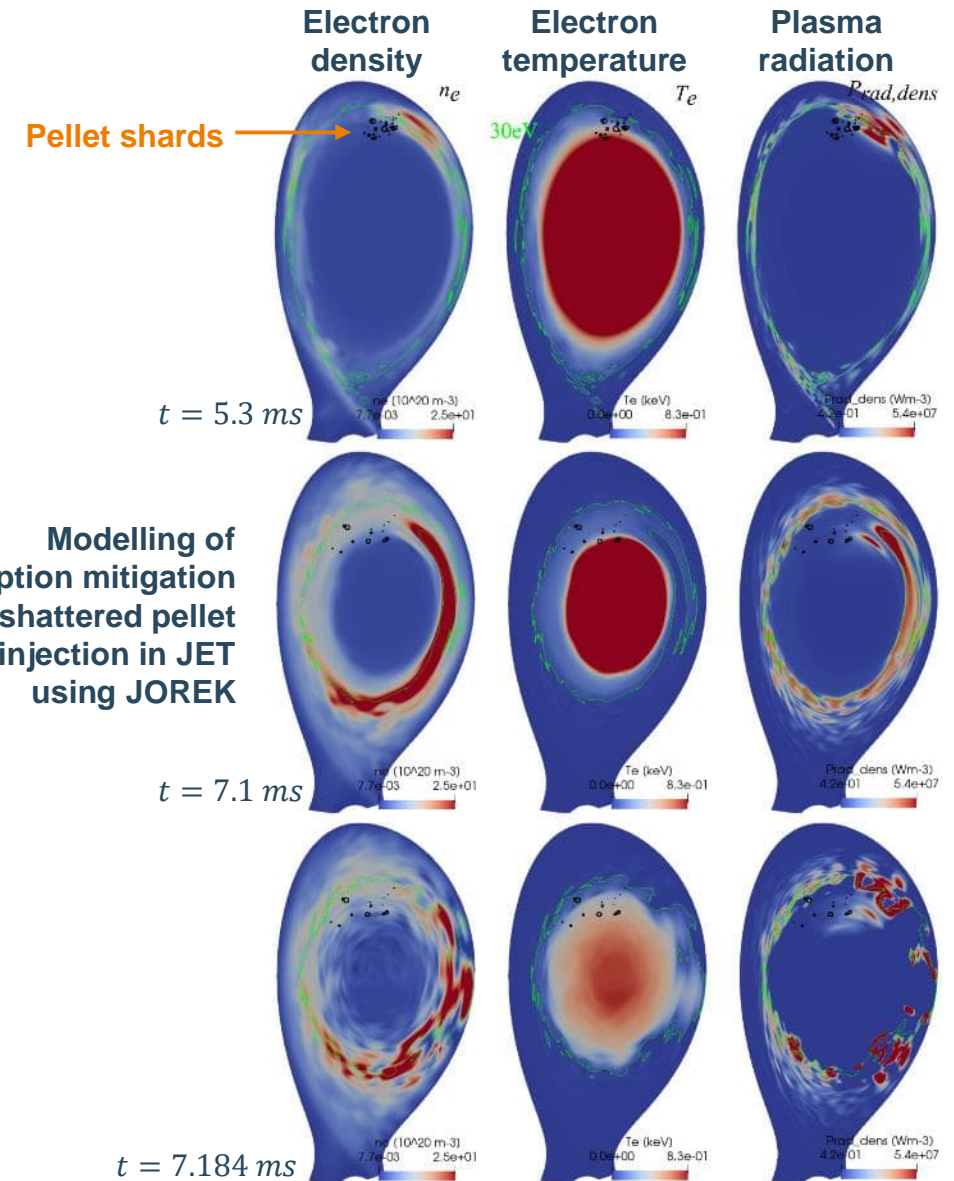
Melting of first wall in JET due to runaway electrons

I. Jecu, IAEA Fusion Energy Conference 2023



- **Solutions**

- Plasma operation avoiding resonant surfaces or eliminating them
- Detection of instabilities
- Mitigation as last resort



Challenges in MCF – power handling

Power balance

$$P_{fus} + P_{ext} = P_{loss} + P_{rad} + \frac{dW}{dt}$$

- α particles are confined and heat plasma
- Radiation and particle losses result in heat fluxes to the wall

Expected heat fluxes to first wall

G. Federici, Fusion Eng. Des. (2015) / M. Wischmeier, J. Nucl. Mater. (2015)

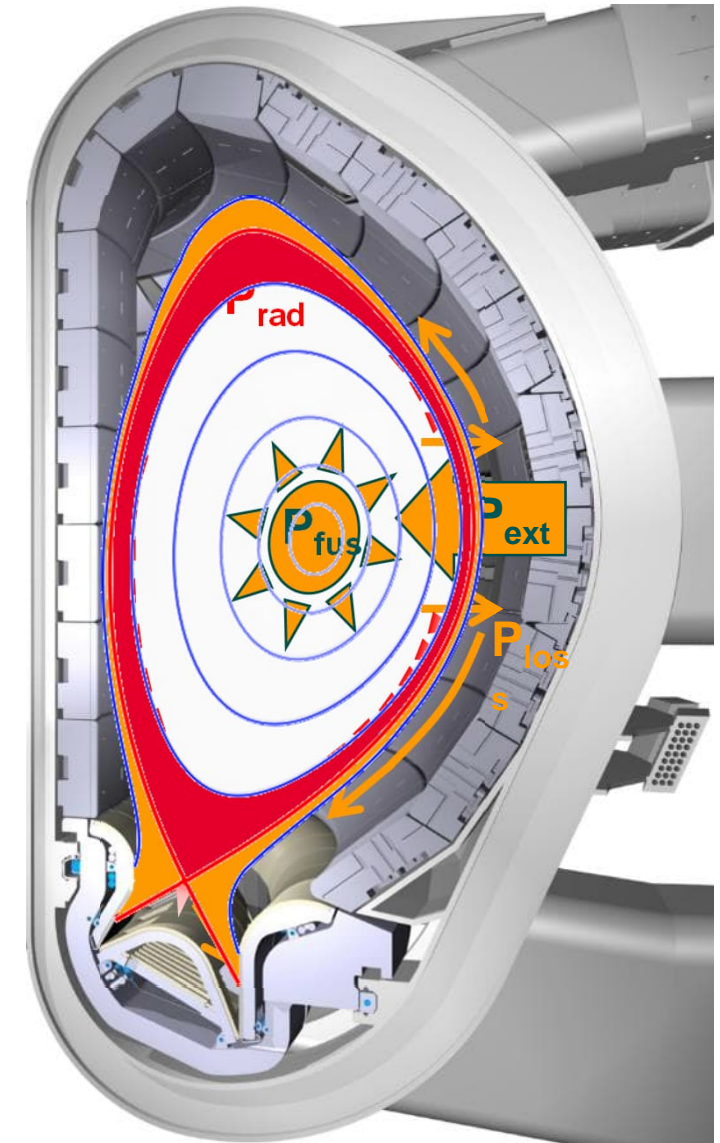
| | ITER | DEMO |
|--------------------------|-----------------------|------------------------|
| Unmitigated power fluxes | 50 MW/m ² | ≥300 MW/m ² |
| Material limit | ~10 MW/m ² | ~5 MW/m ² |

⇒ Increase power dissipation to more than 95%

Solution: impurity seeding

≥ 30% in SOL & divertor

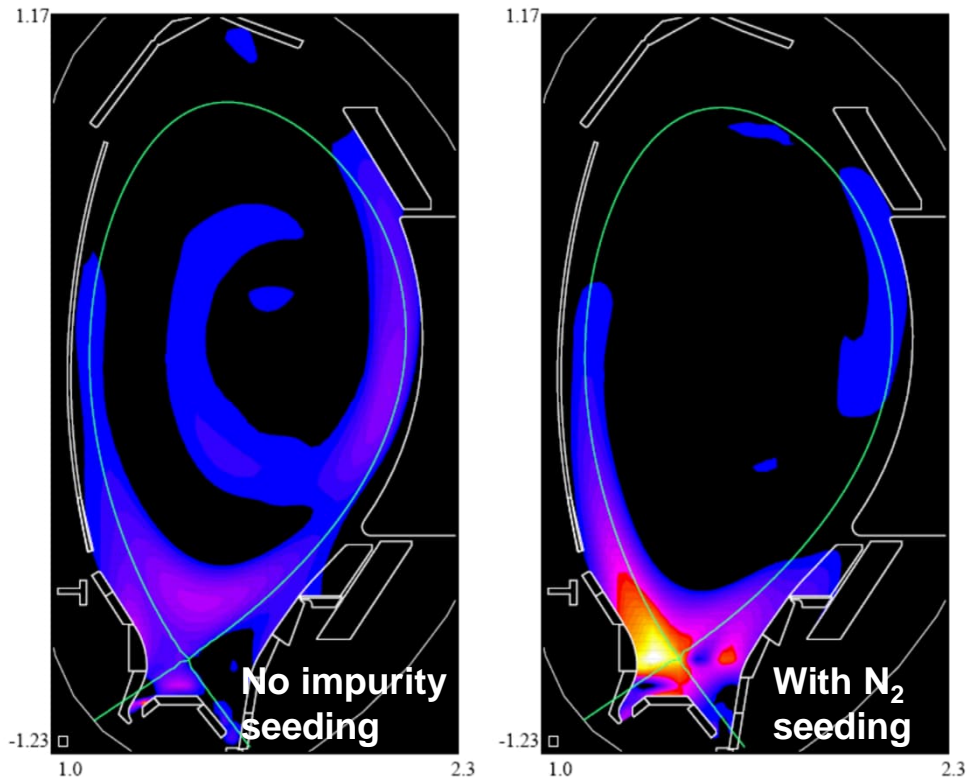
≤ 65% inside confined region, minimal central radiation



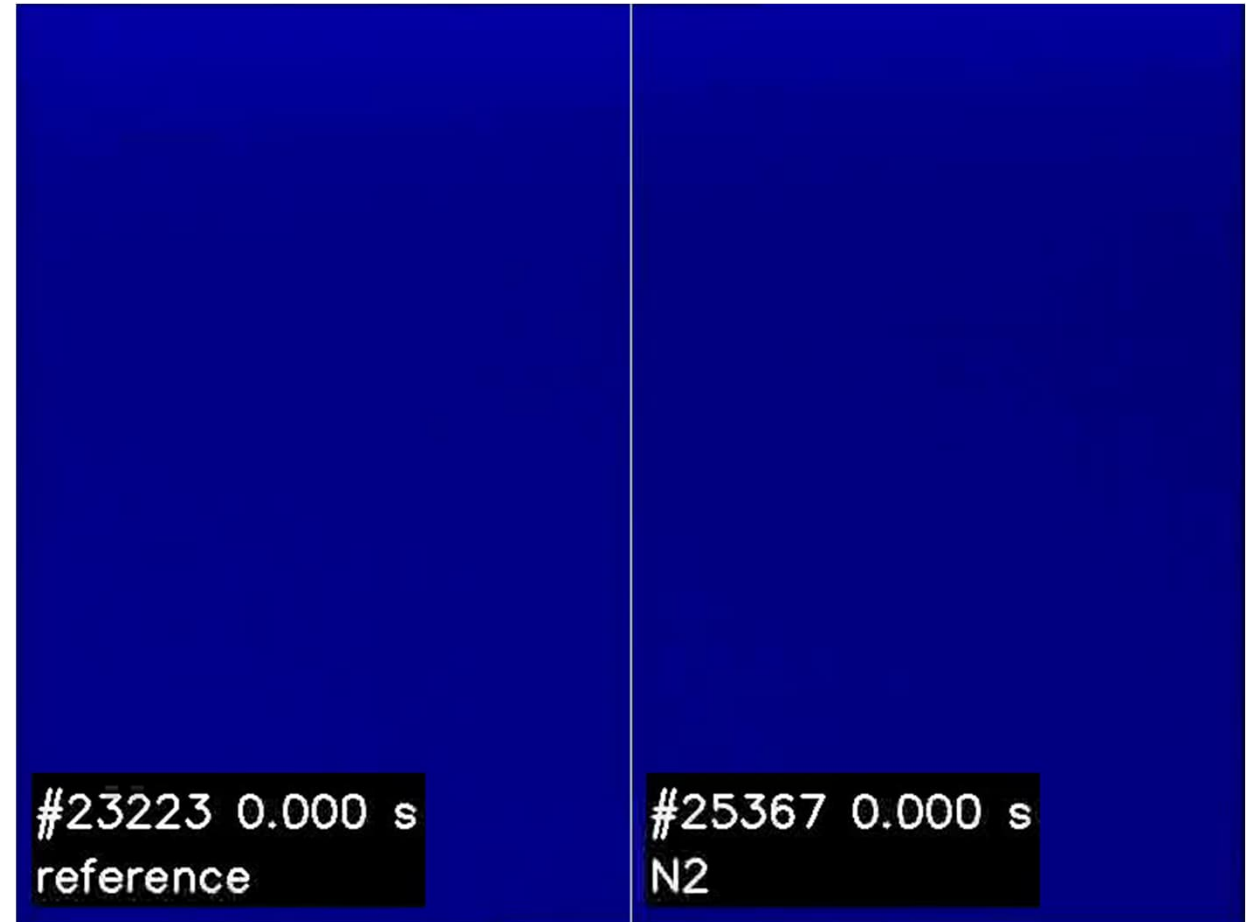
Challenges in MCF – power handling

Results from ASDEX Upgrade

Bolometry of total radiated power



Thermography of power load to divertor target





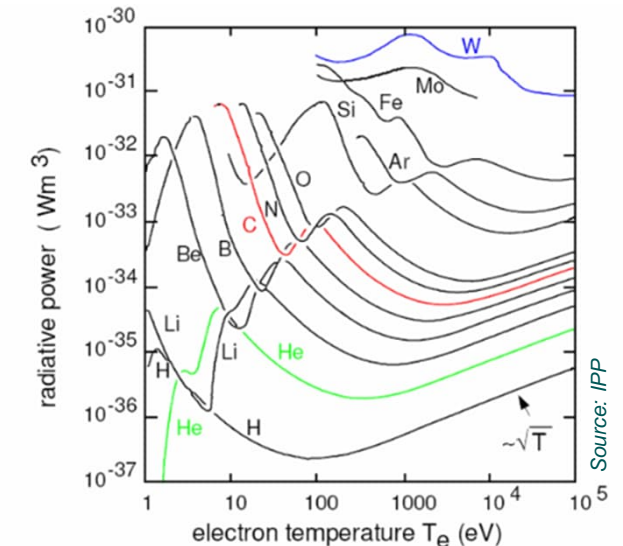
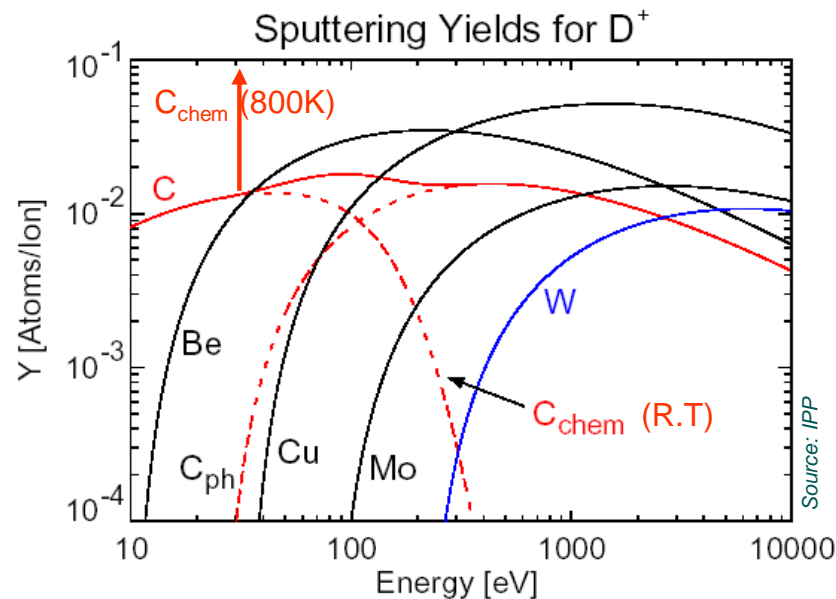
Challenges in MCF – the wall material

Particle and heat fluxes from plasma interact with first wall, which needs to provide

- Thermal stability
- Low erosion
- Low retention of T

Solution: W (tungsten)

- Highest melting point of all solids
- Very low erosion yield
- Very strong radiator
→ only a fraction of $\leq 10^{-4}$ can be tolerated in plasma centre



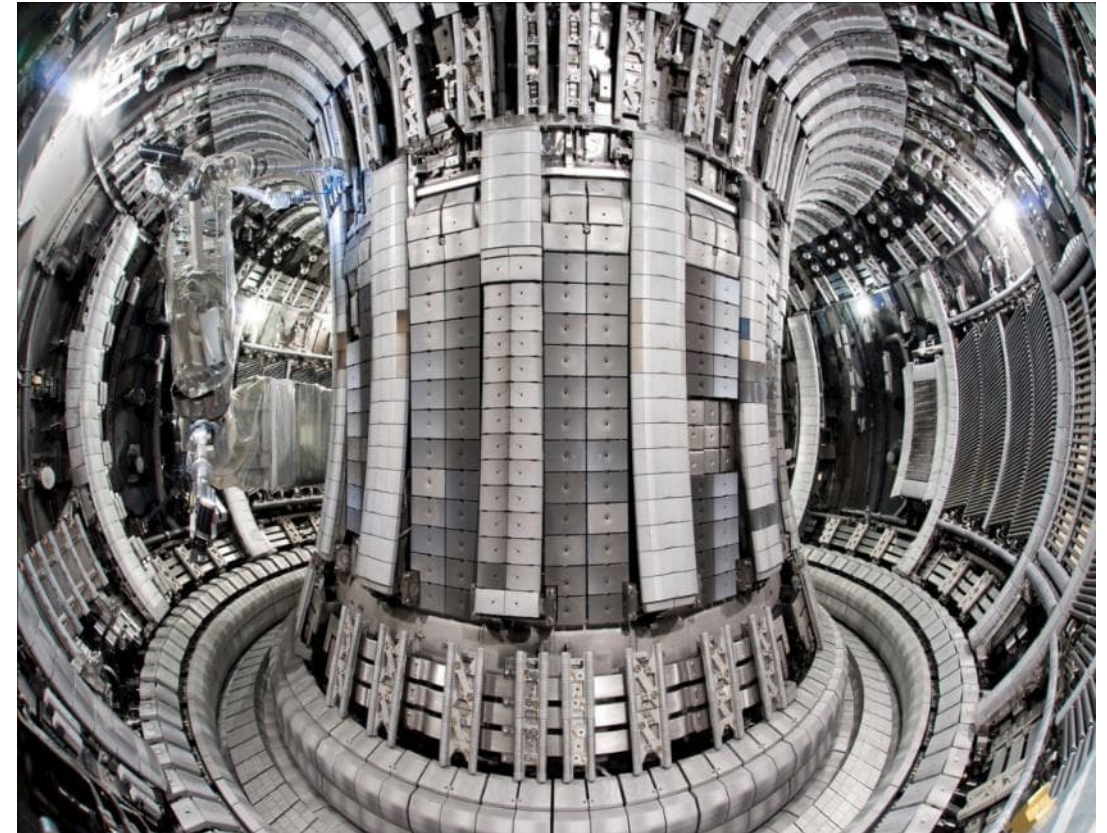
Challenges in MCF – the wall material

ASDEX Upgrade



Since 2007: complete first wall out of W

JET

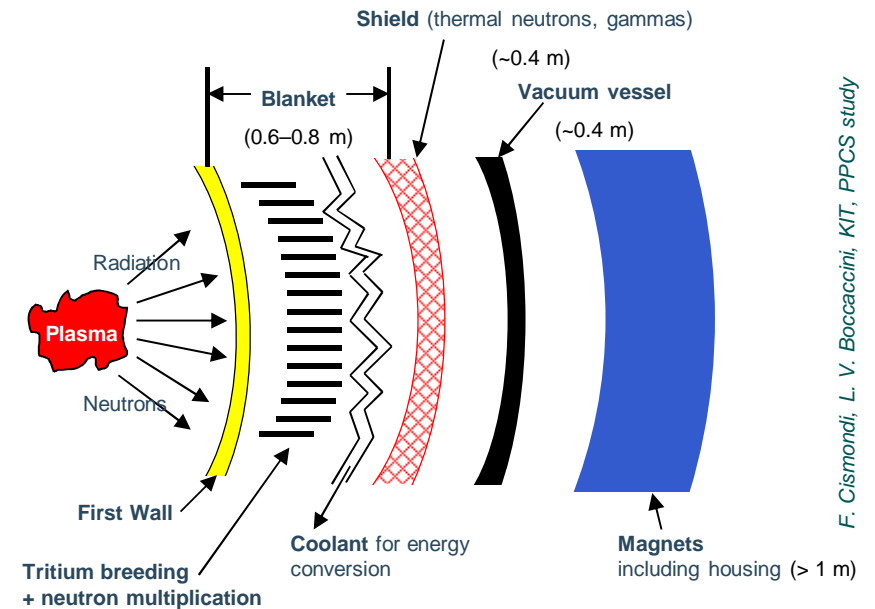
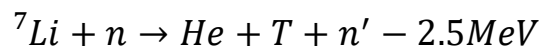
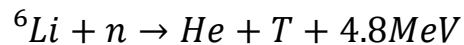
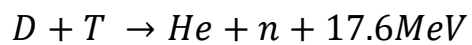
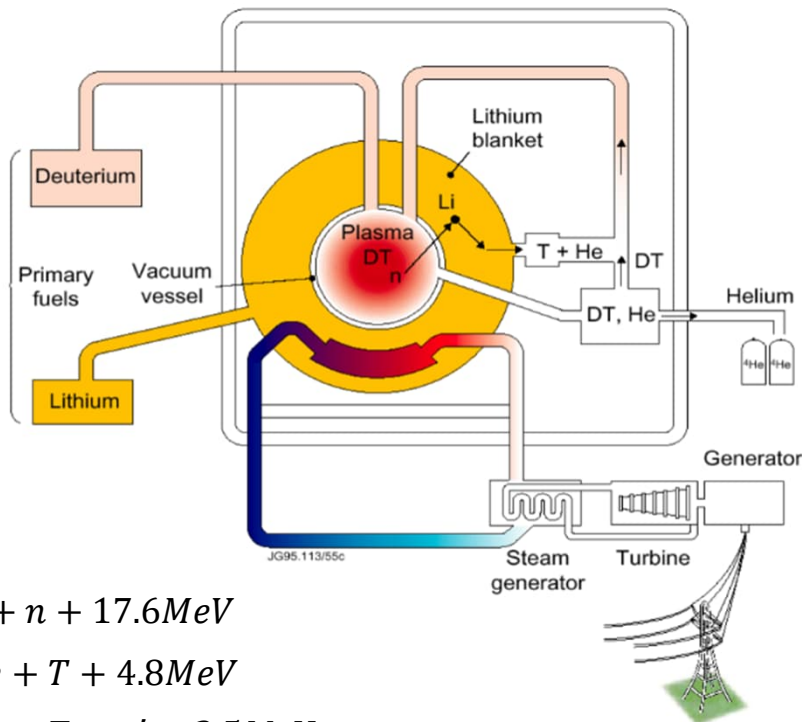


Since 2011: W-divertor

Challenges in MCF – tritium breeding

Tritium decays quickly ($T_{1/2} = 12.3 \text{ a}$)

➤ Produce T as part of fuel cycle



F. Cismondi, L. V. Boccaccini, KIT, PPCS study

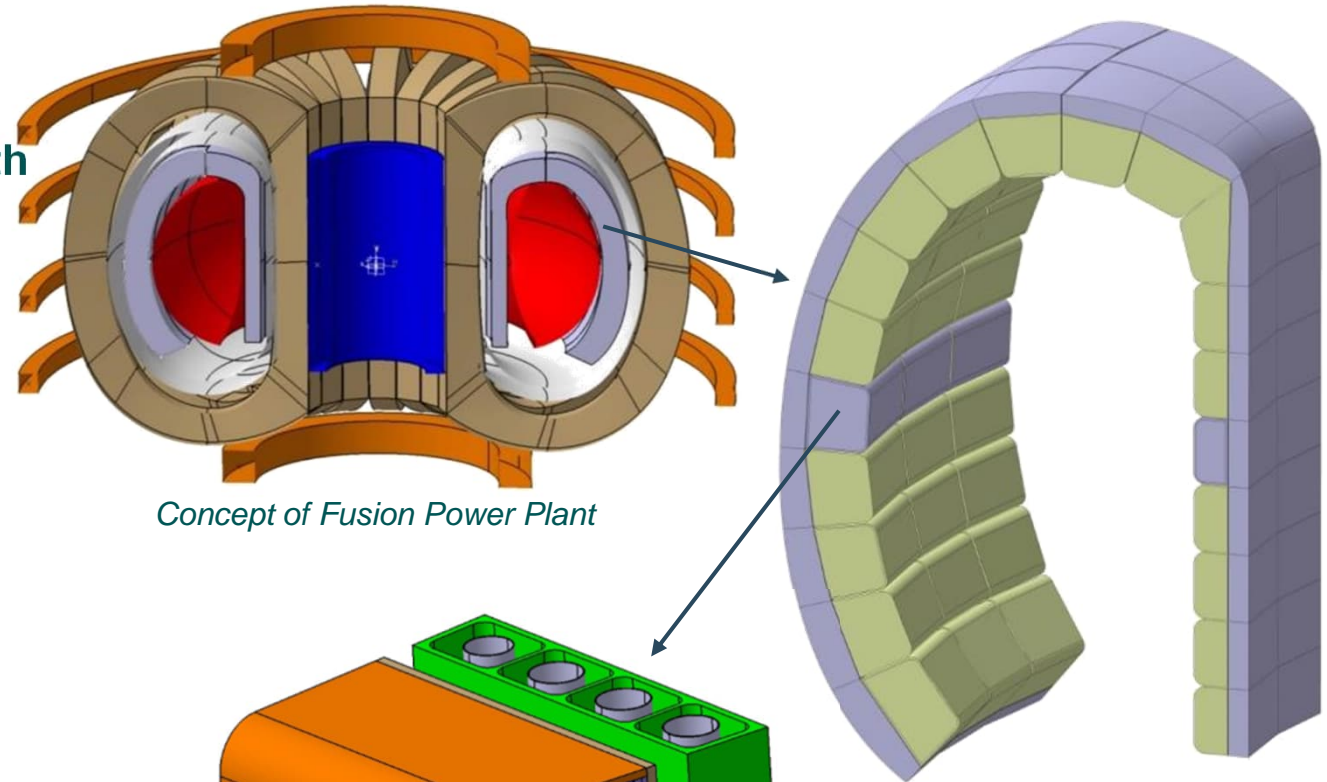
- To produce sufficient amount of T
 - Tritium breeding rate $TBR > 1.1$
 - Neutron multiplication required
 - Blanket Modules (BM) need to cover as much surface around plasma as possible
 - Thickness of blanket is fixed → determines size

Challenges in MCF – tritium breeding

Coverage with blanket modules competes with access to plasma

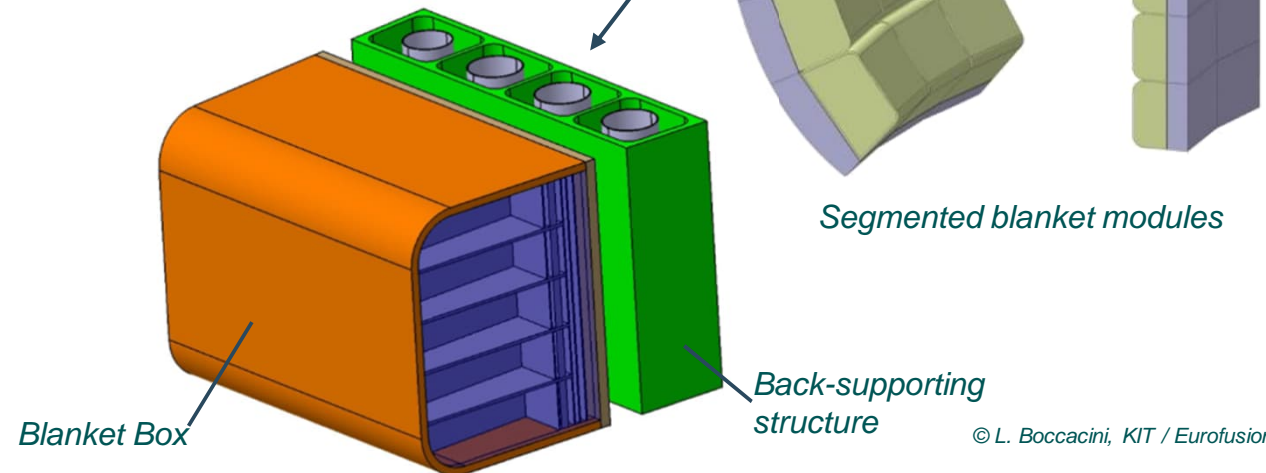
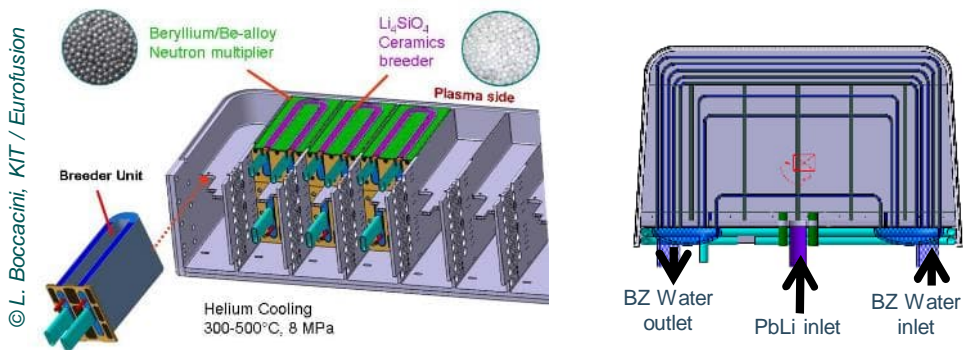
- Integrate external heating into BMs
- Strongly reduced access for diagnostics
- Advanced control schemes required

Main European concepts for BM



He-cooled Pebble Bed (HCPB)

Water-cooled Li-Lead (WCLL)

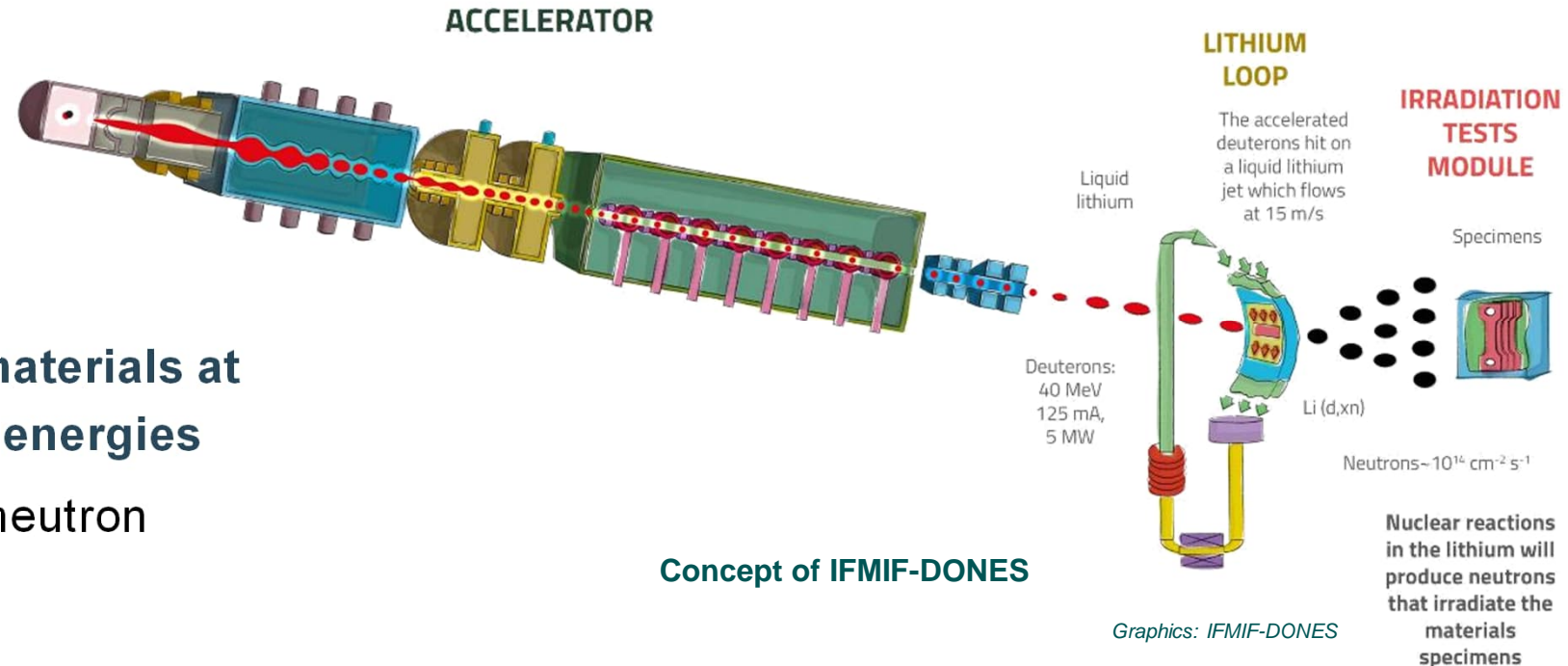


© L. Boccacini, KIT / Eurofusion

Challenges in MCF – structural materials

Neutrons from fusion reactions activate structural materials

- Embrittlement and reduced yield stress
- Development of dedicated materials
 - low-activation steel EUROFER
 - W-fibre reinforced W or Cu
- Testing and verification of materials at relevant neutron fluxes and energies
 - IFMIF-DONES – dedicated neutron test facility for fusion



Challenges in MCF – structural materials

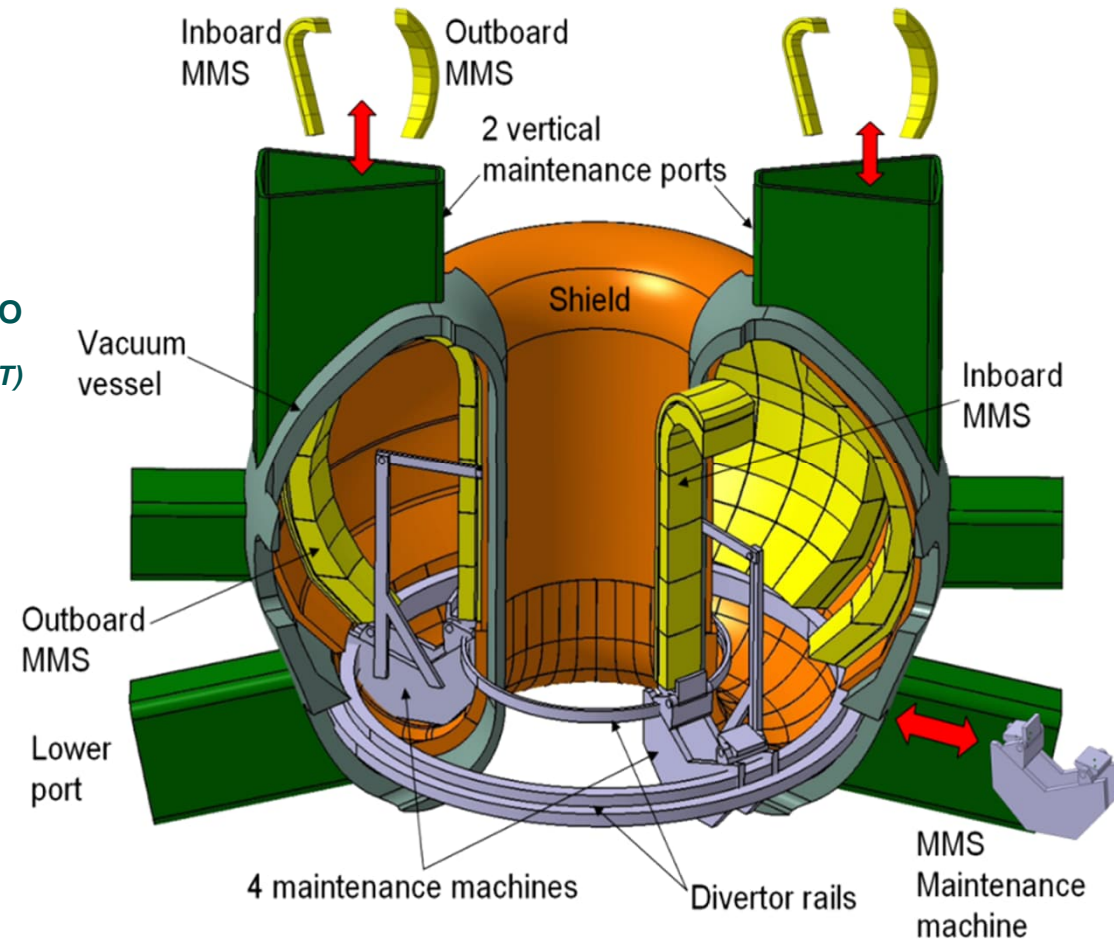
Activation prevents human access

- Maintenance and repair through remote-handling

Remote handling concept for blanket modules in DEMO

courtesy D. Filsinger, L. V. Boccaccini, et al. (KIT)

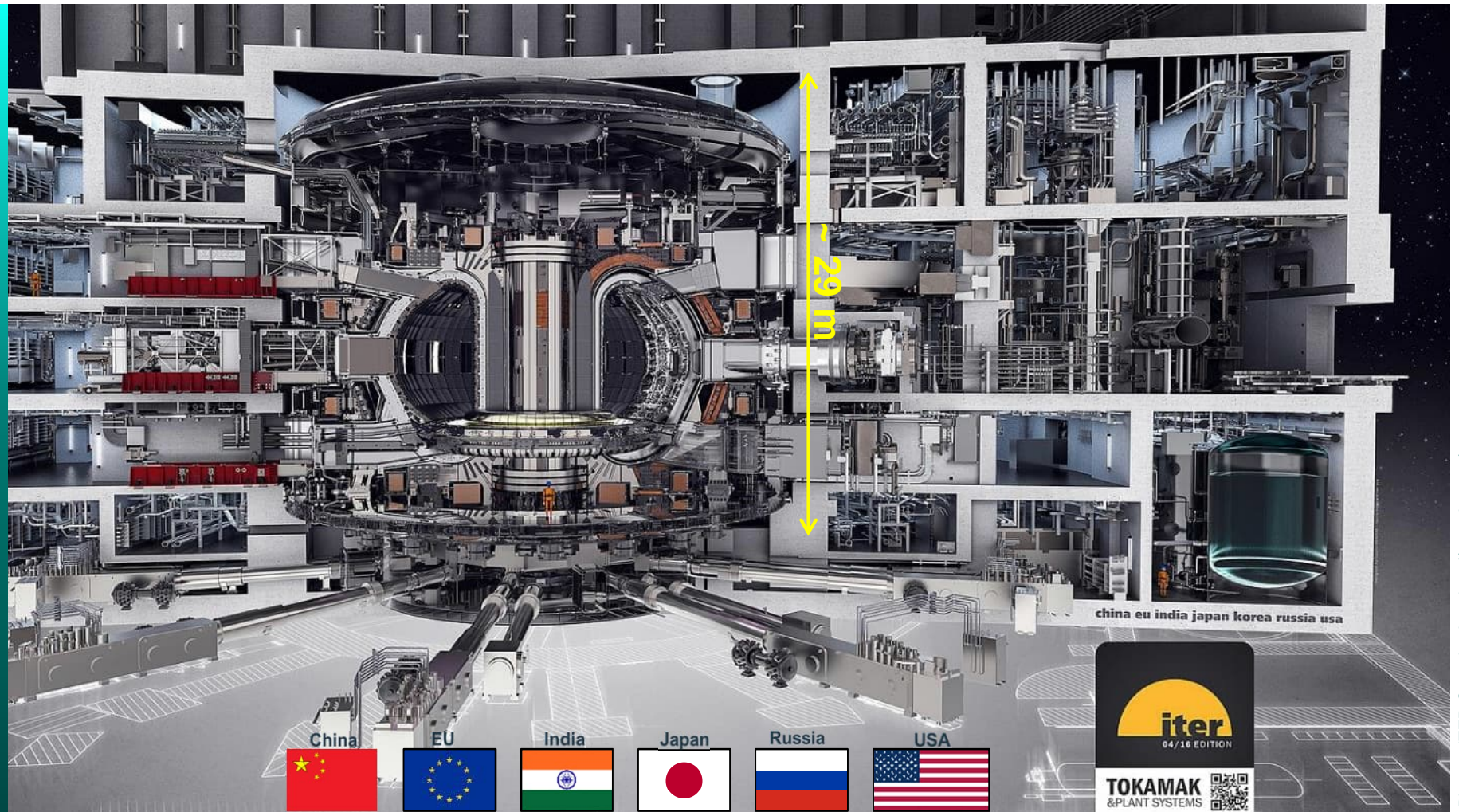
Low activated materials can be handled after few decades of storage, but large volumes of waste will be produced and handling needs to be regulated



ITER – the largest tokamak ever built

Objectives

- Create more energy than needed to heat plasma ($Q > 1$)
- Create plasma dominantly heated by fusion reactions
- Achieve $Q = 10$ at plasma current of 15 MA
- Test technologies, materials and physics for commercial fusion-based electricity



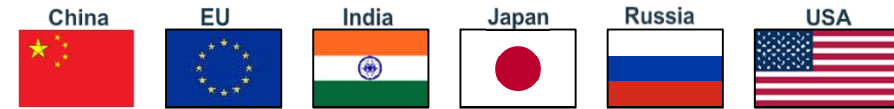
Picture: ITER Organization, <http://www.iter.org/>



ITER – a worldwide fusion experiment

Seven international partners

- Partners constitute > 80% of world GDP
- Contributions to ITER are provided „in kind“
- Budget is essentially with partners



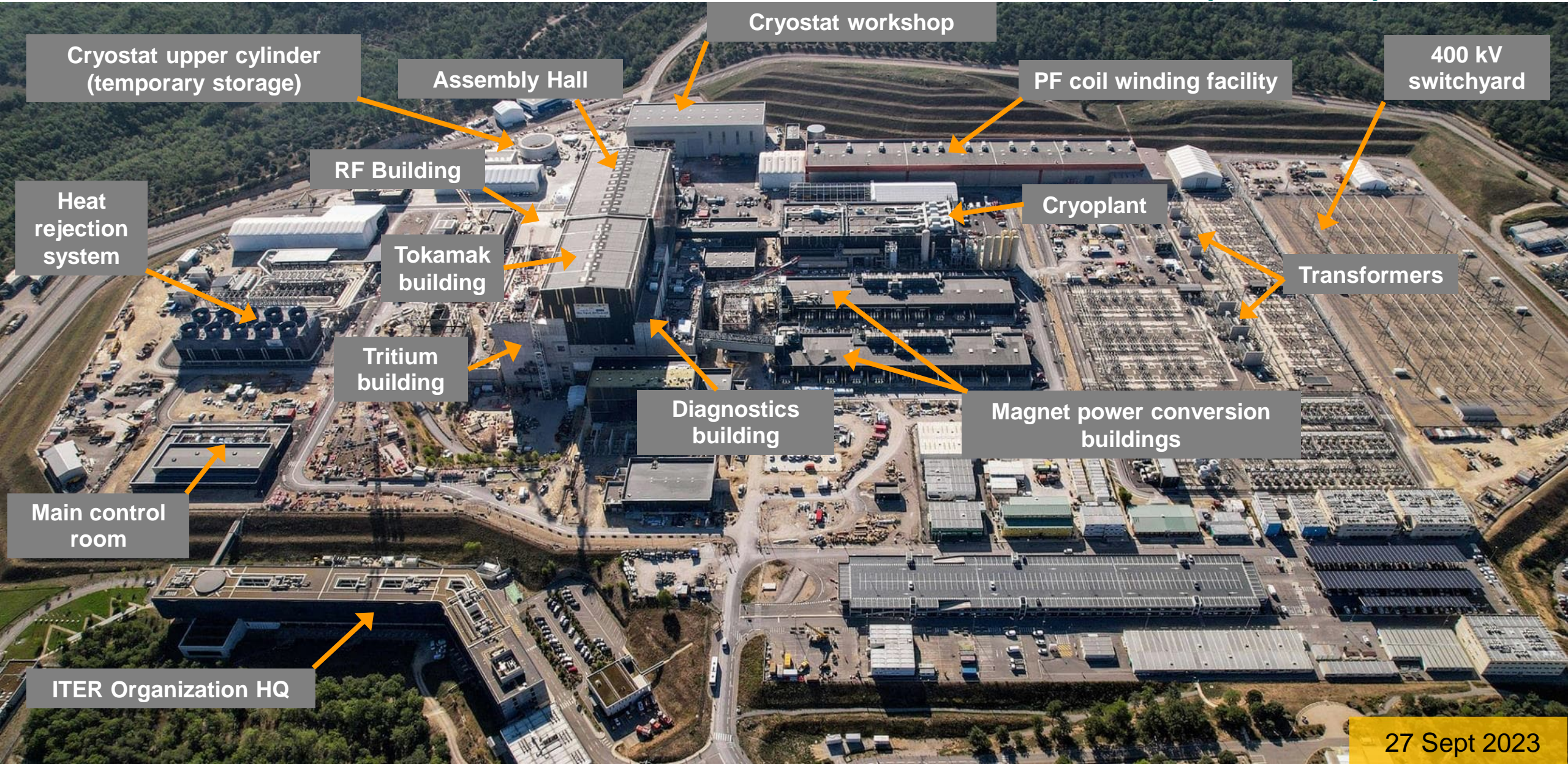
ITER Organization

- Legal entity
- Overall integrator of the project
- Nuclear operator of the ITER facility

ITER site in France



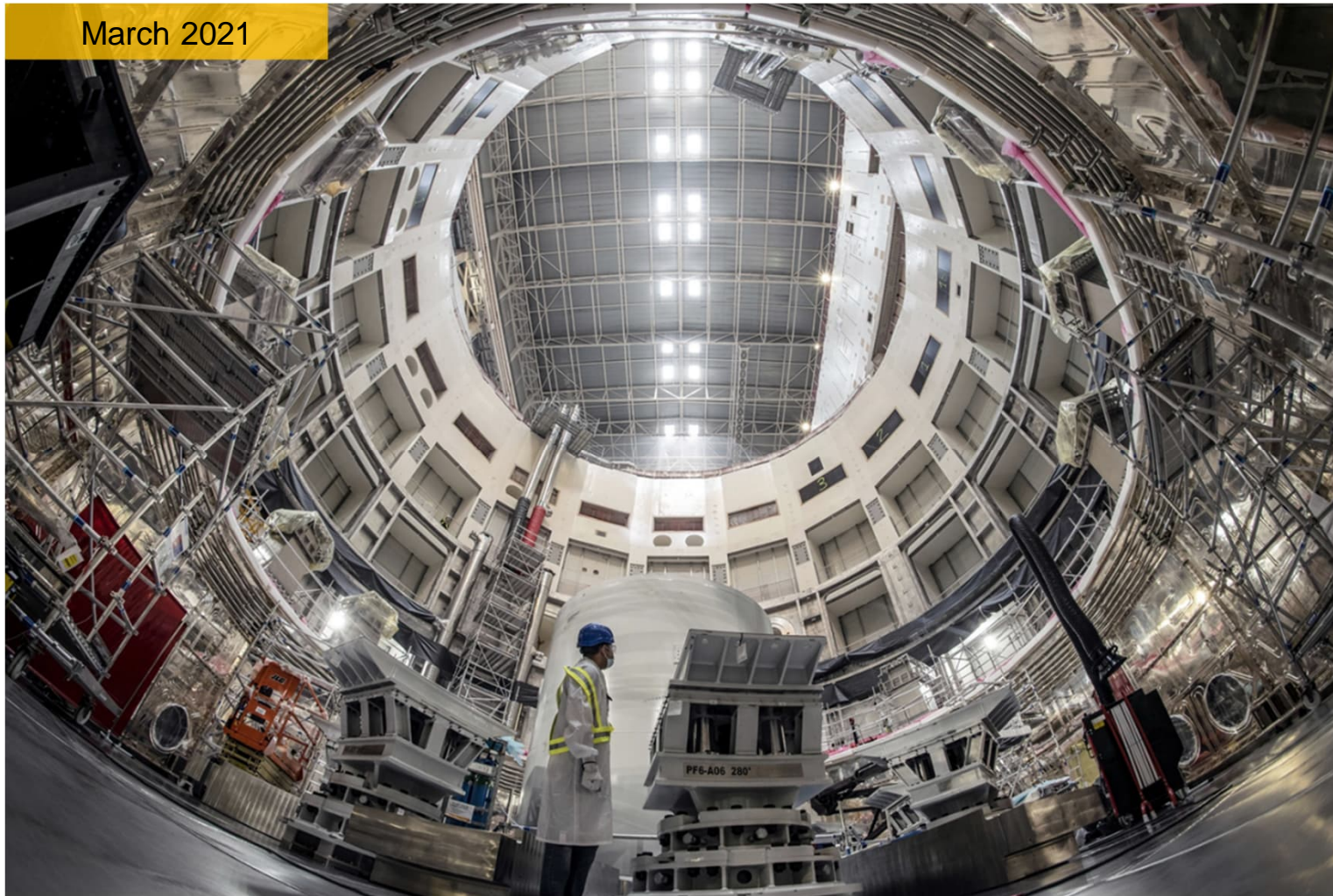
Picture: ITER Organization, <http://www.iter.org/>





ITER construction

March 2021



Picture: ITER Organization, <http://www.iter.org/>

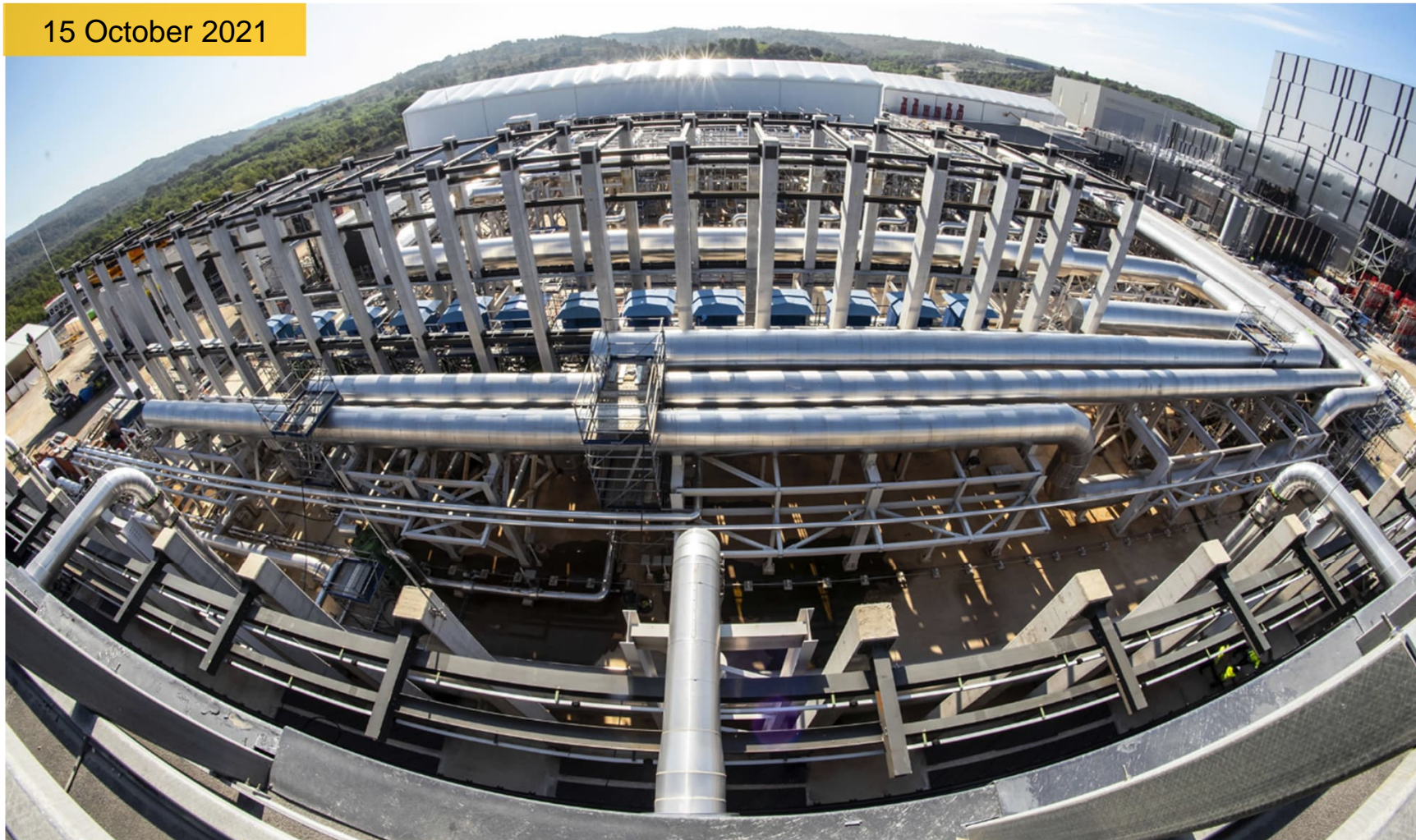
Tokamak Pit

- Shielded area that eventually will house the tokamak

ITER construction



15 October 2021



Cooling plant

**Optimise
management of an
intermittent demand
(peak: 1145 MW,
between pulses: 160
MW)**

Picture: ITER Organization, <http://www.iter.org>

ITER construction



25 November 2021



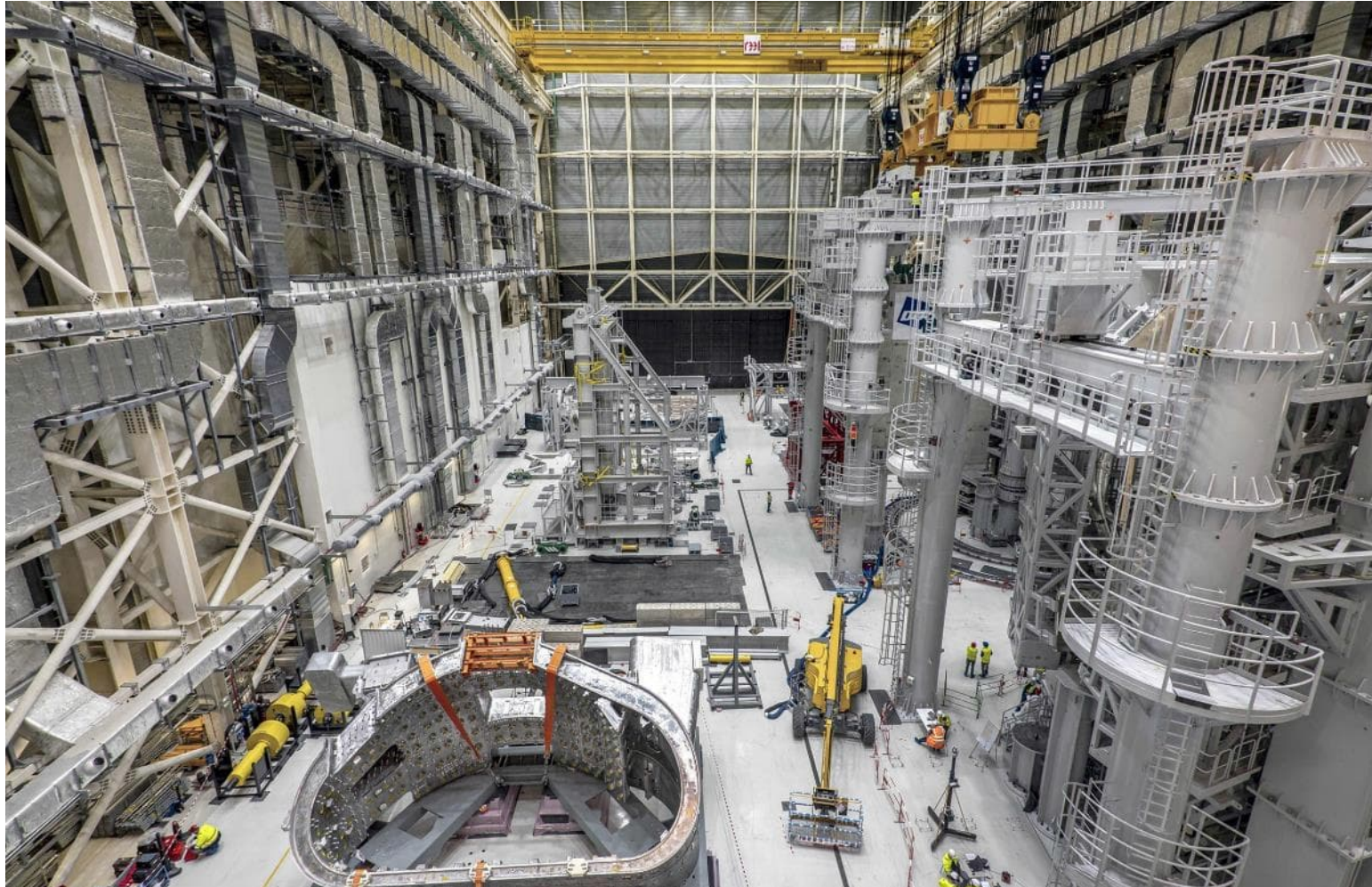
Cryoplant

- Largest single unit plant in the world.
- Liquid He inventory 25 t
- Power:
 - 75 kW at 4.5 K (He)
 - 1300 kW at 80 K (N₂)

Picture: ITER Organization, <http://www.iter.org/>



ITER construction – Assembly and installation of first complete VV sector



28 February 2021

- Assembly Hall being prepared for first VV Sector upending



ITER construction – Assembly and installation of first complete VV sector



26 March 2021

- Start of upending sequence to place Sector 6 into the vertical position



ITER construction – Assembly and installation of first complete VV sector

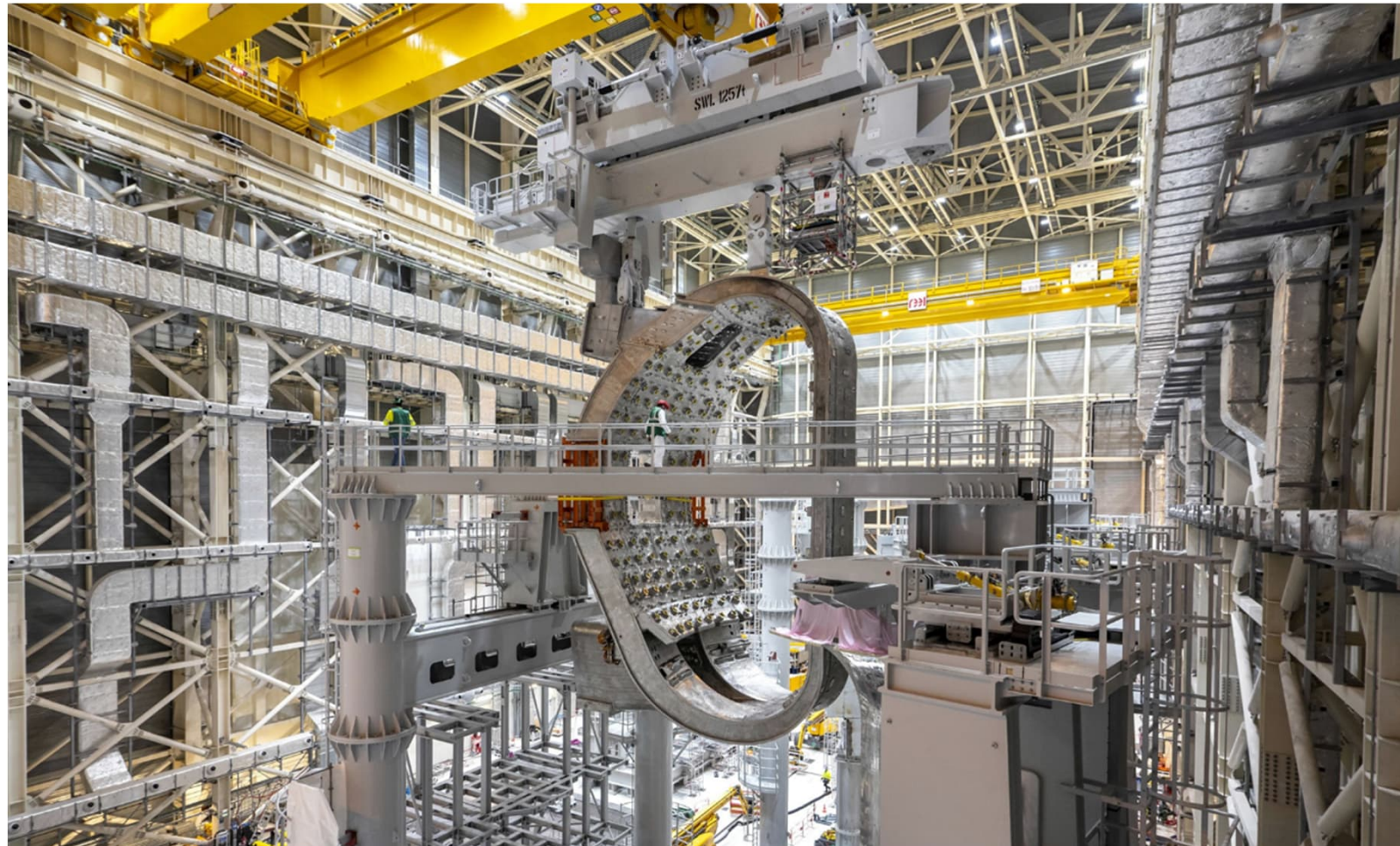


6 April 2021

- Final step: transfer to the Sub-Sector Assembly Tool (SSAT)



ITER construction – Assembly and installation of first complete VV sector

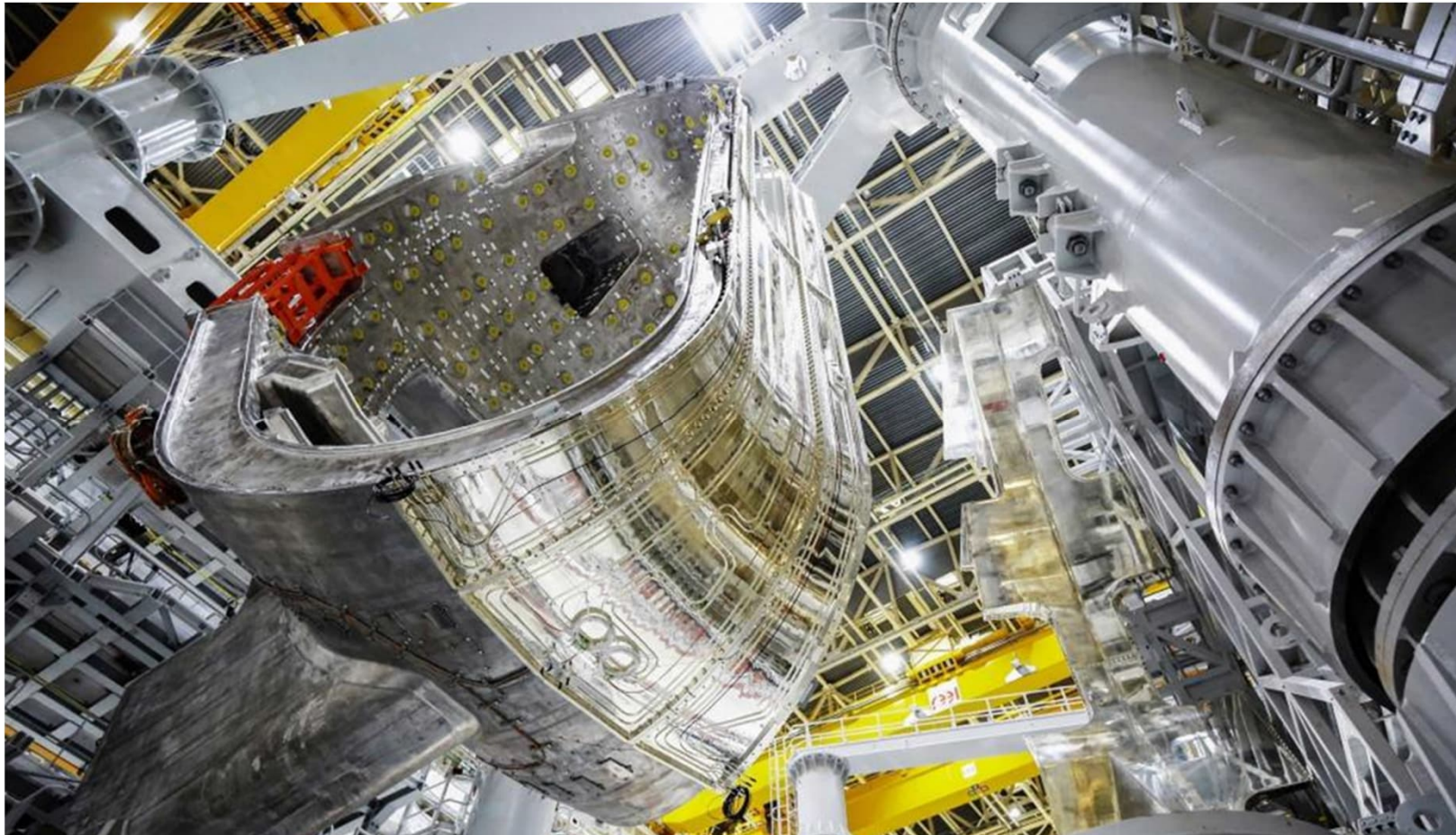


6 April 2021

- And finally, inserted from above into the SSAT



ITER construction – Assembly and installation of first complete VV sector

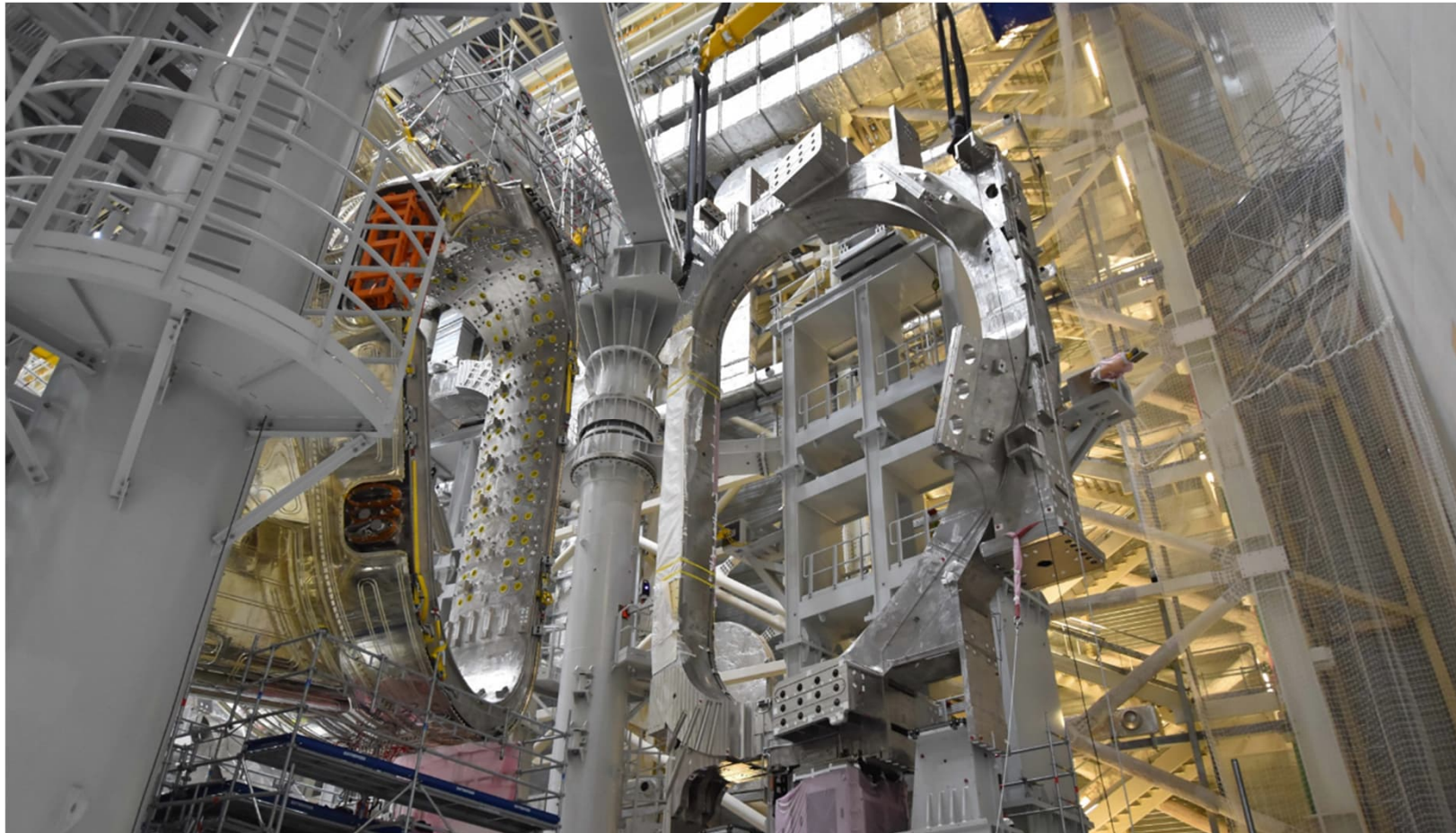


21 June 2021

- Inboard thermal shield section clamped to Sector 6



ITER construction – Assembly and installation of first complete VV sector

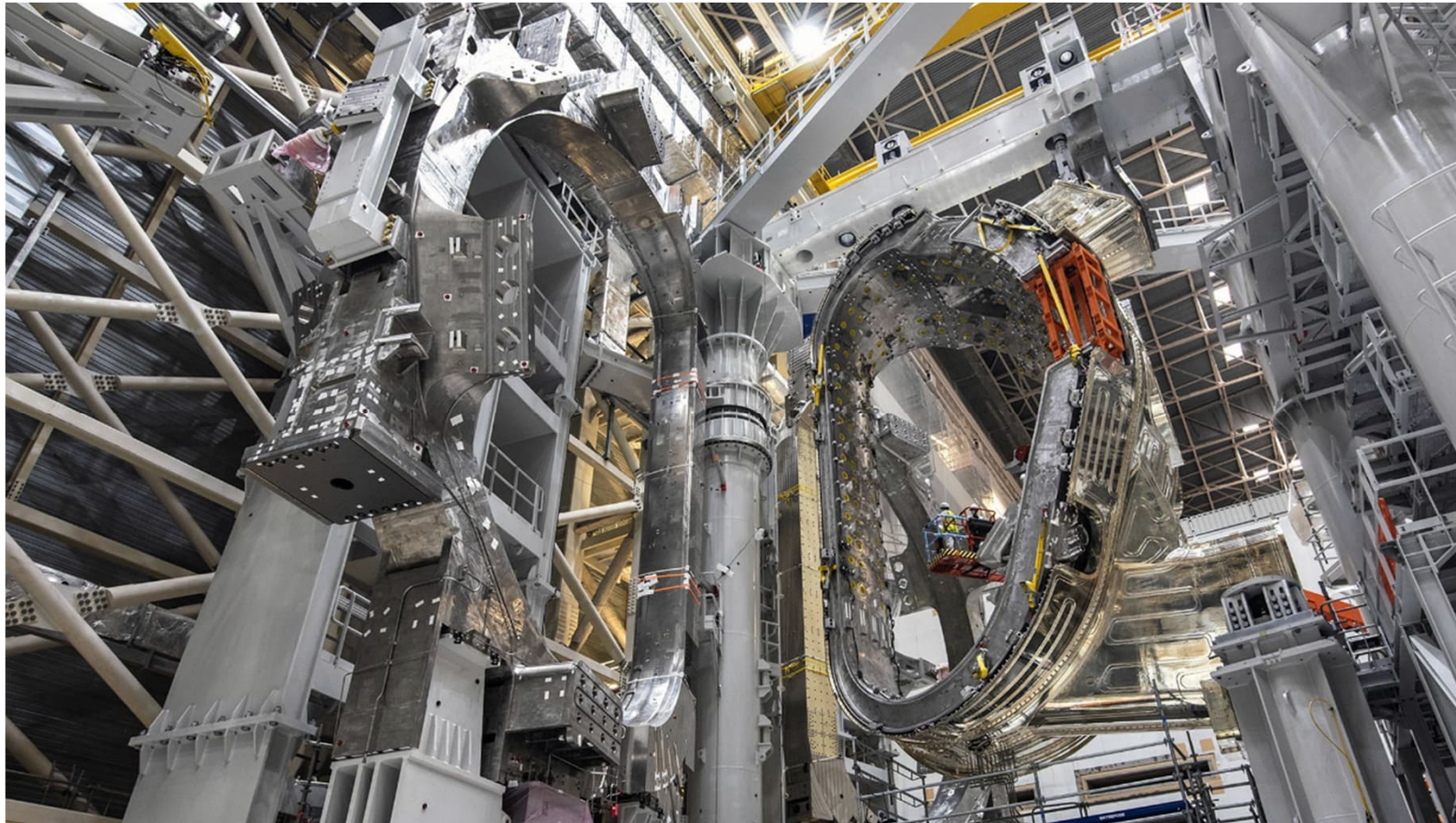


12 August 2021

- Full VV Sector 6 with thermal shield on and Toroidal Field coil 12 mounted on SSAT



ITER construction – Assembly and installation of first complete VV sector

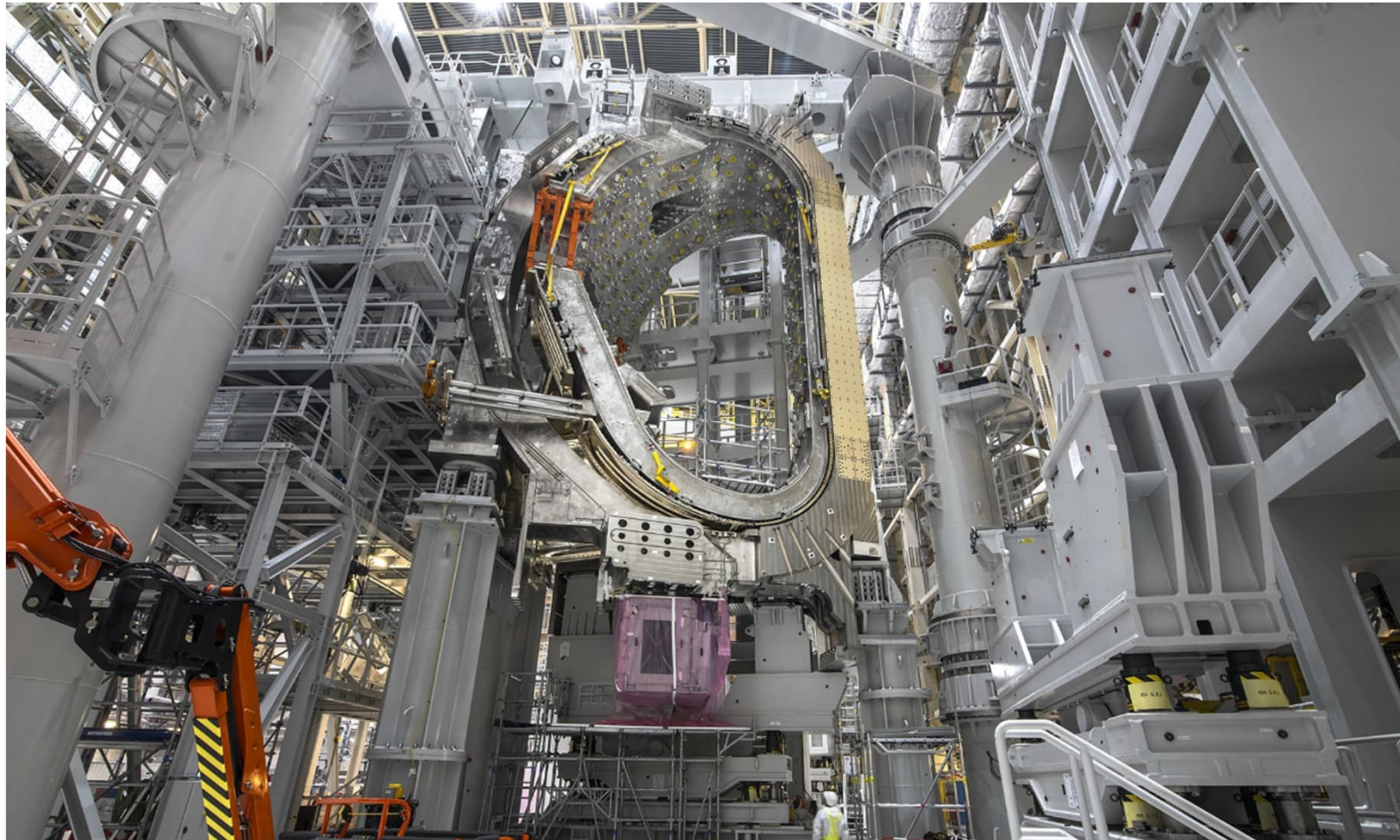


30 August 2021

- Just prior to final positioning of TF coils



ITER construction – Assembly and installation of first complete VV sector

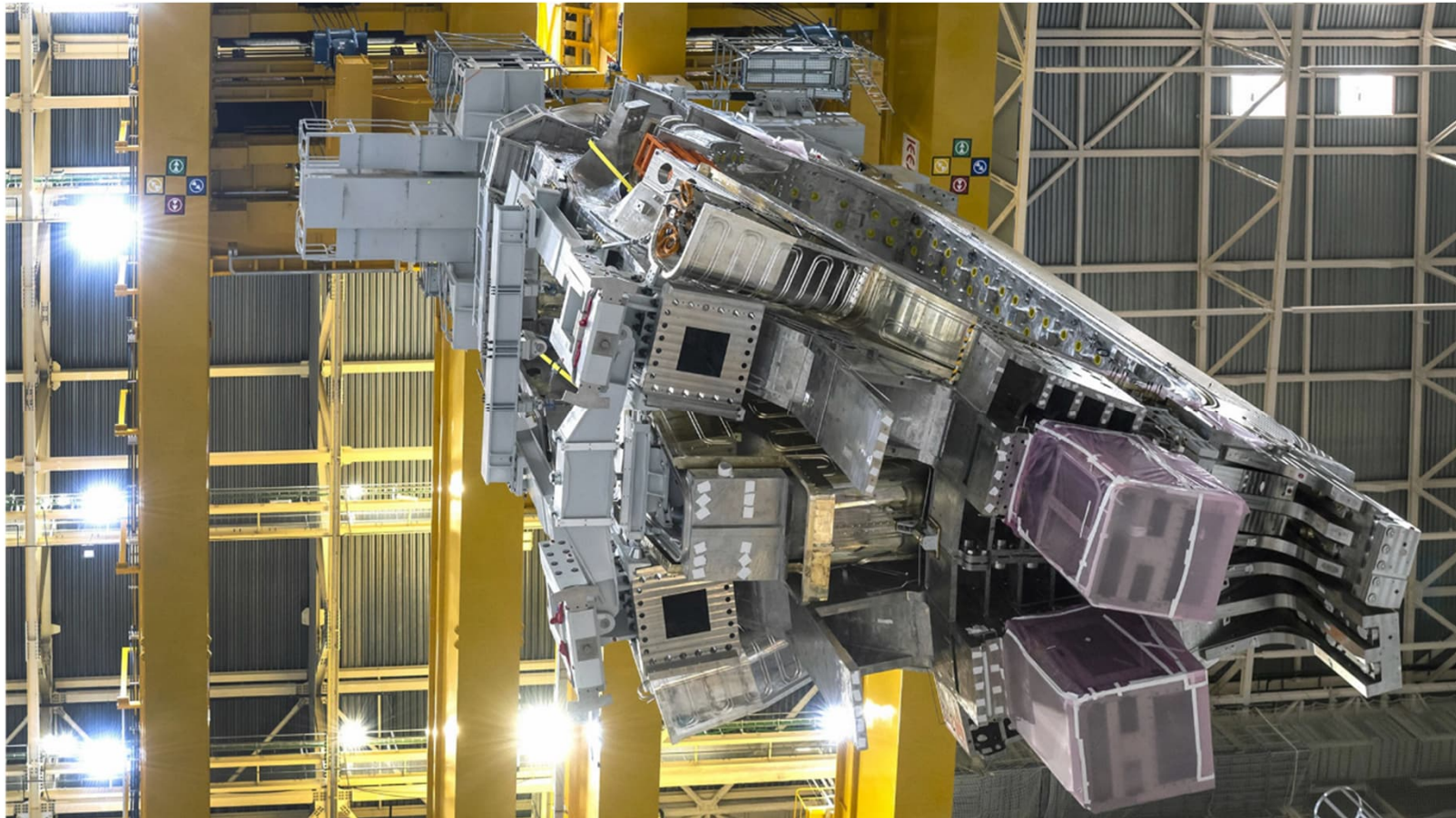


15 November 2021

- First complete Sector 6 ready for pit installation



ITER construction – Lift and placement of first sector



11 May 2022

- Sector 6 fully lifted out of SSAT-2 and rotated 90°



ITER construction – Lift and placement of first sector



11 May 2022

- Sector 6 fully lifted out of SSAT-2 and rotated 90°



ITER construction – Lift and placement of first sector



11 May 2022

- Over the tokamak pit wall: 20 cm gap



ITER construction – Lift and placement of first sector

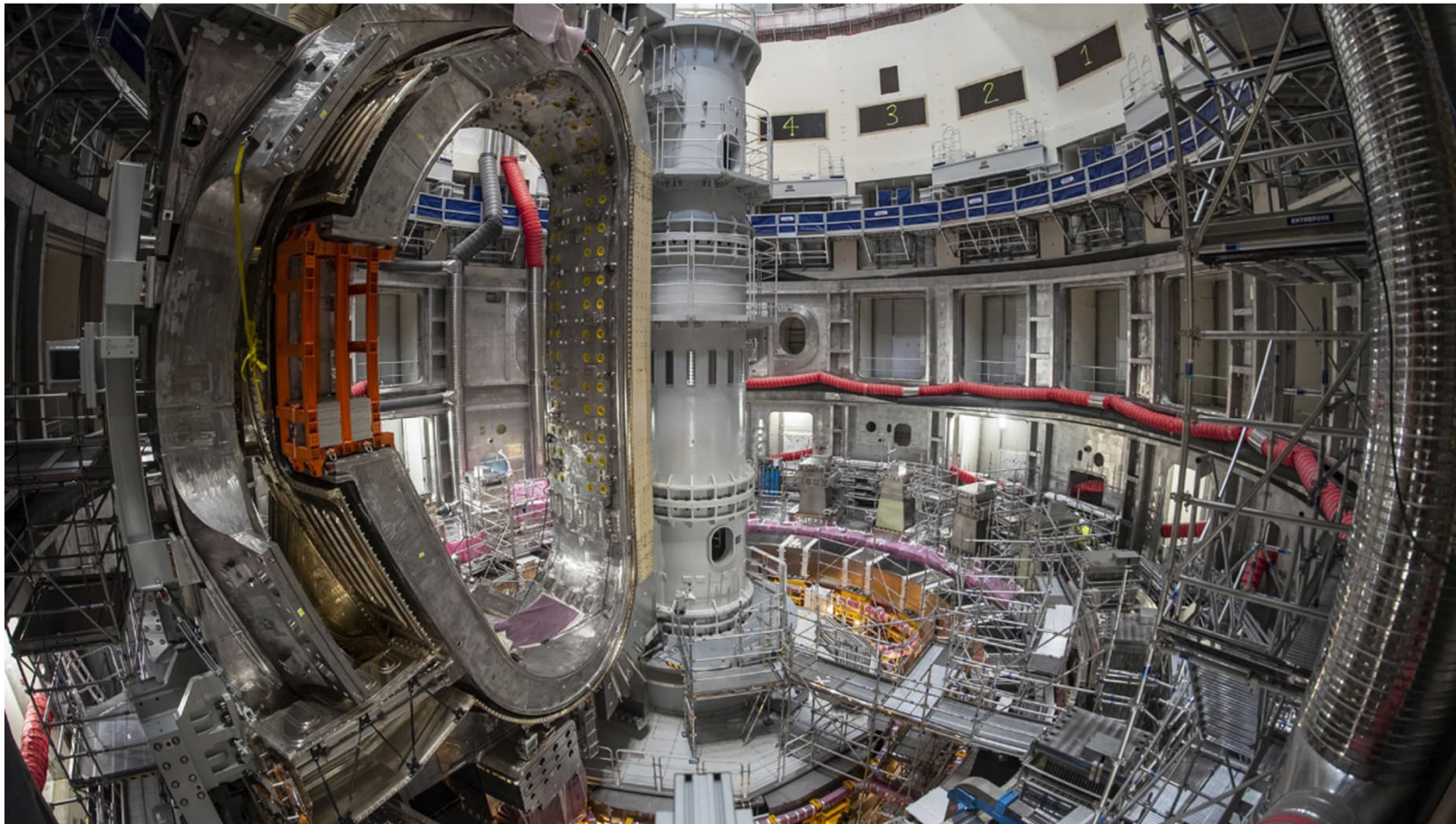


11 May 2022

- Lowering into the tokamak pit



ITER construction – Lift and placement of first sector

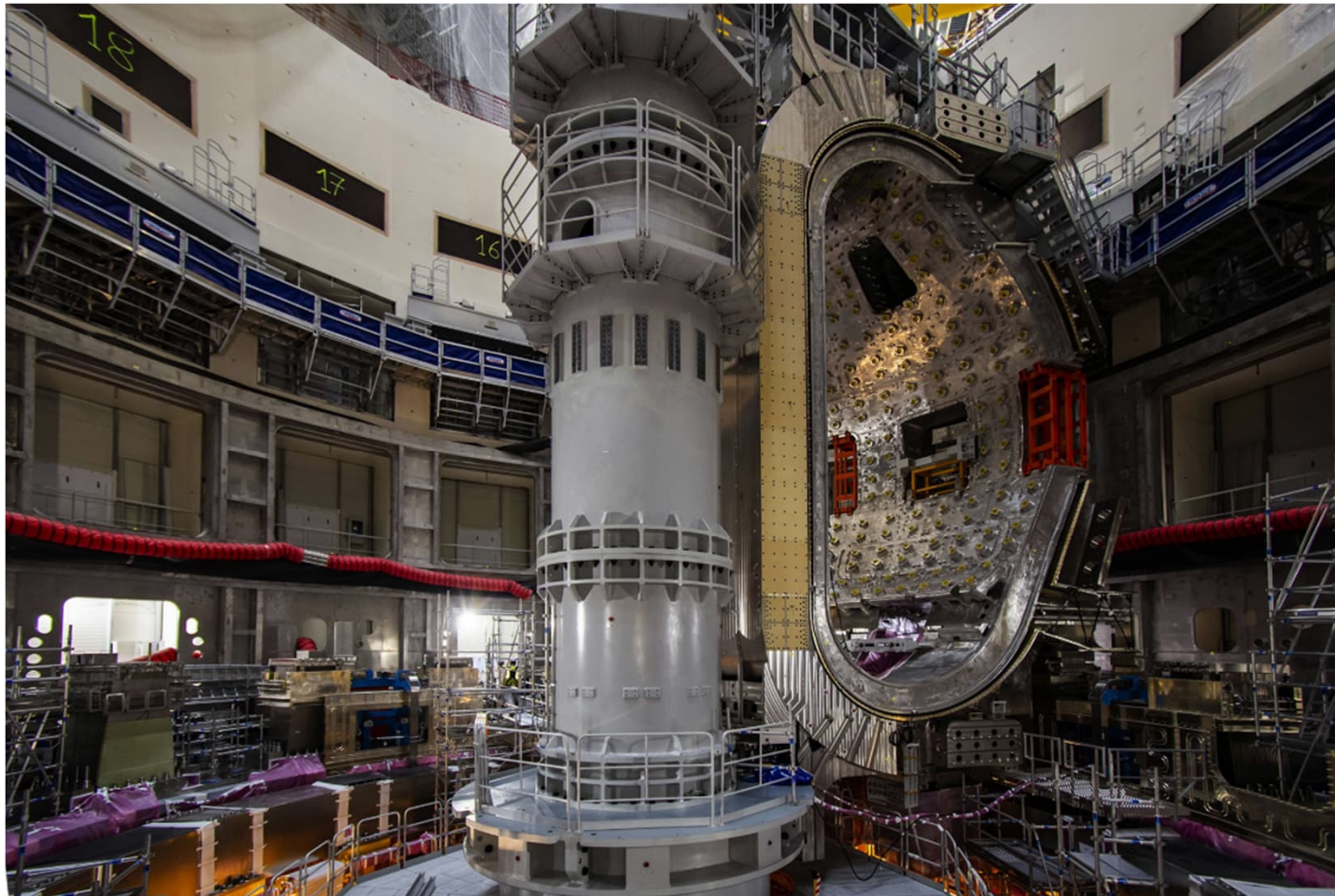


14 June 2022

- In-place and alignment procedure underway



ITER construction – Lift and placement of first sector

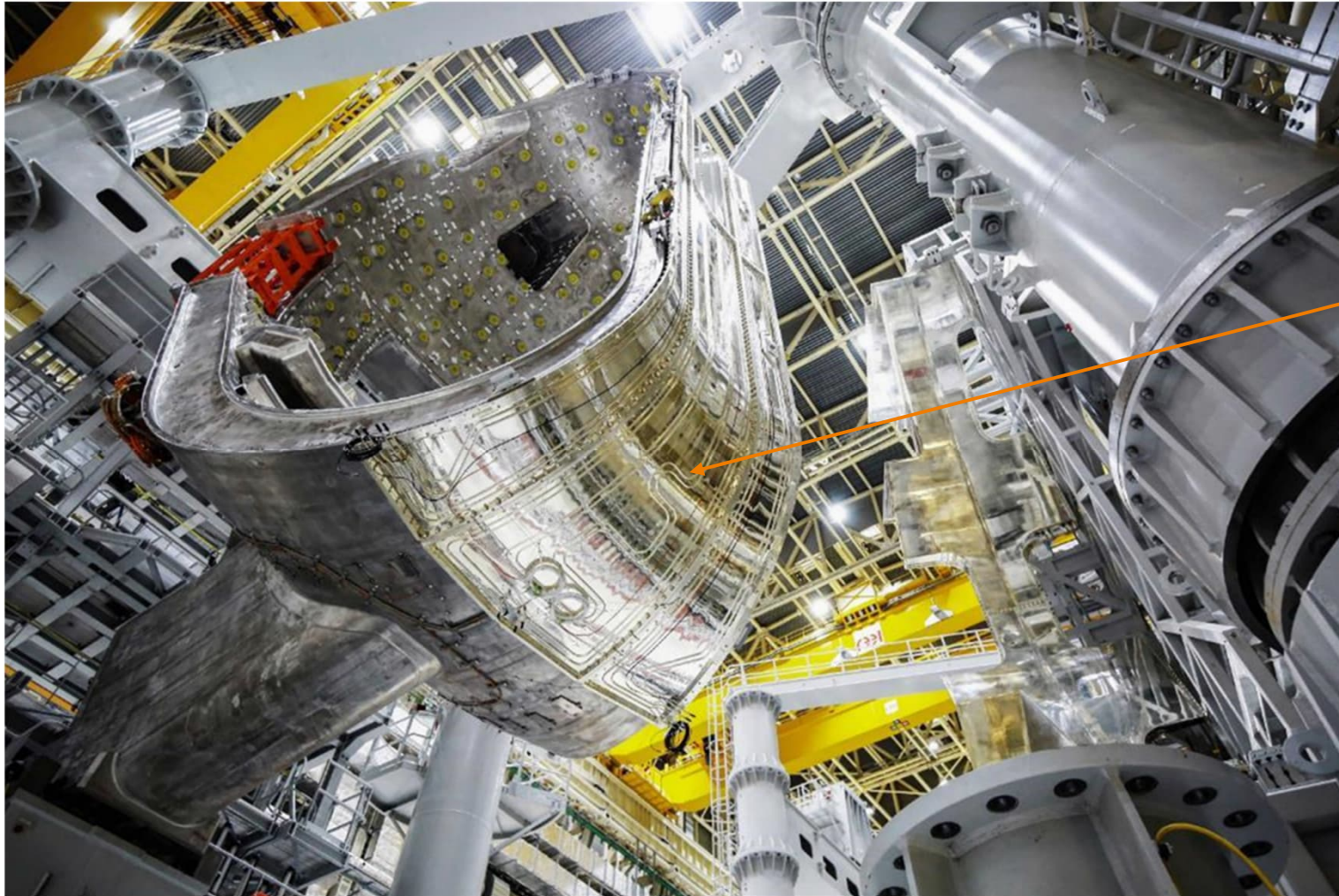


22 September 2022

- Final alignment complete.
- Sector placed to millimetric precision according to physics specifications

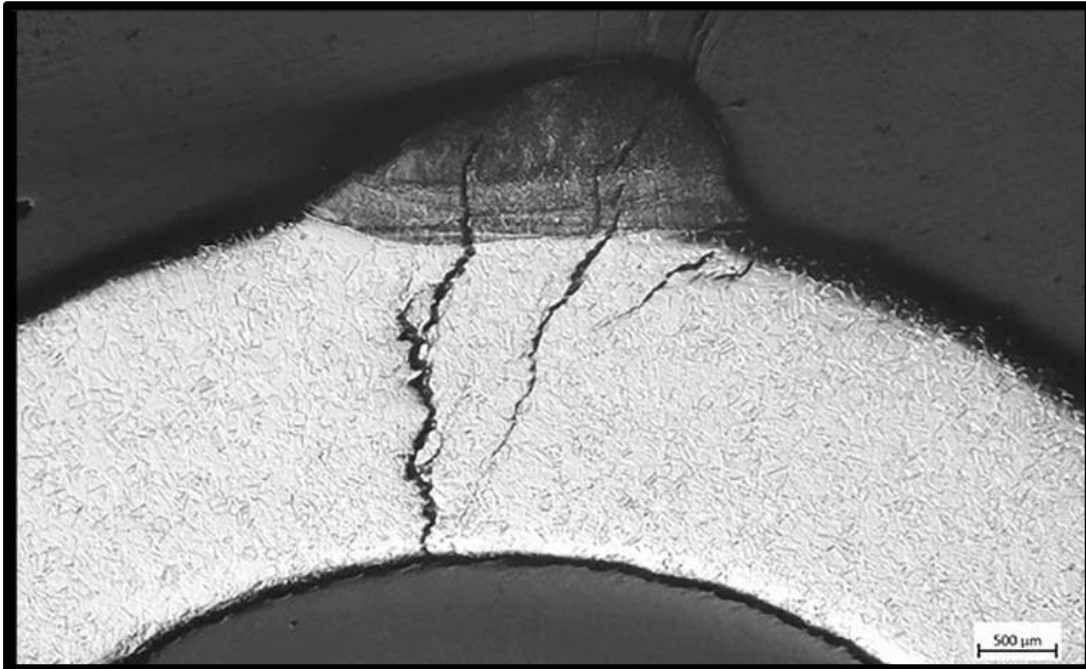


Status of ITER – Stress corrosion cracks in cooling pipes

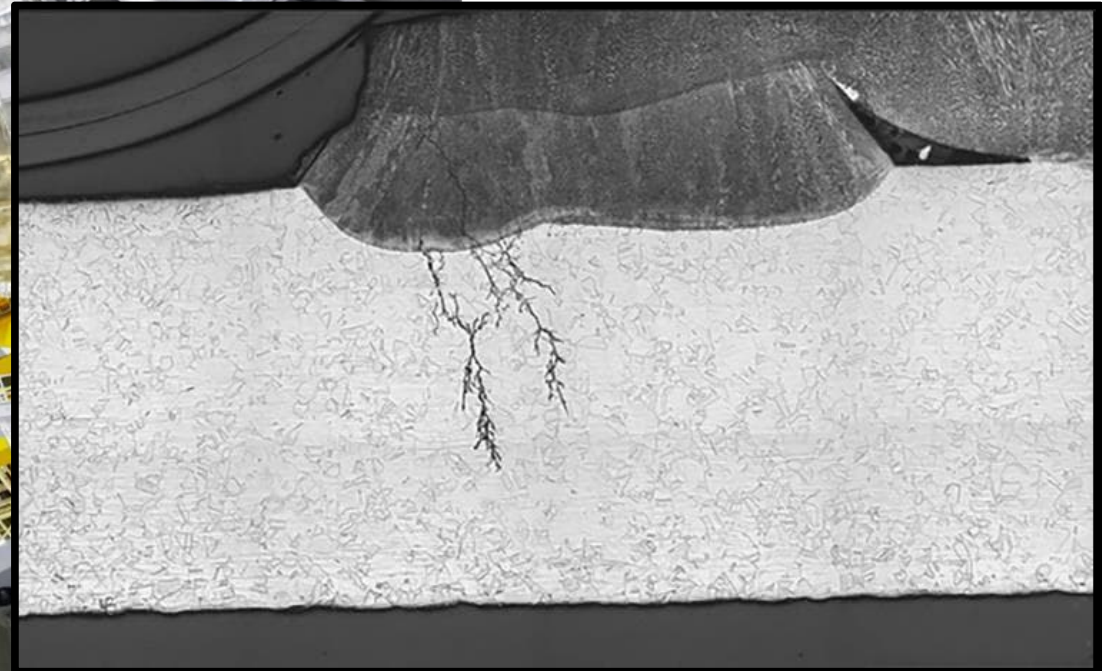


- **Stress corrosion cracks (SCC) on thermal shields: problem found on several VV shields**
- **Assumed to be present in many locations in VV and cryostat shields**

Status of ITER – Stress corrosion cracks in cooling pipes

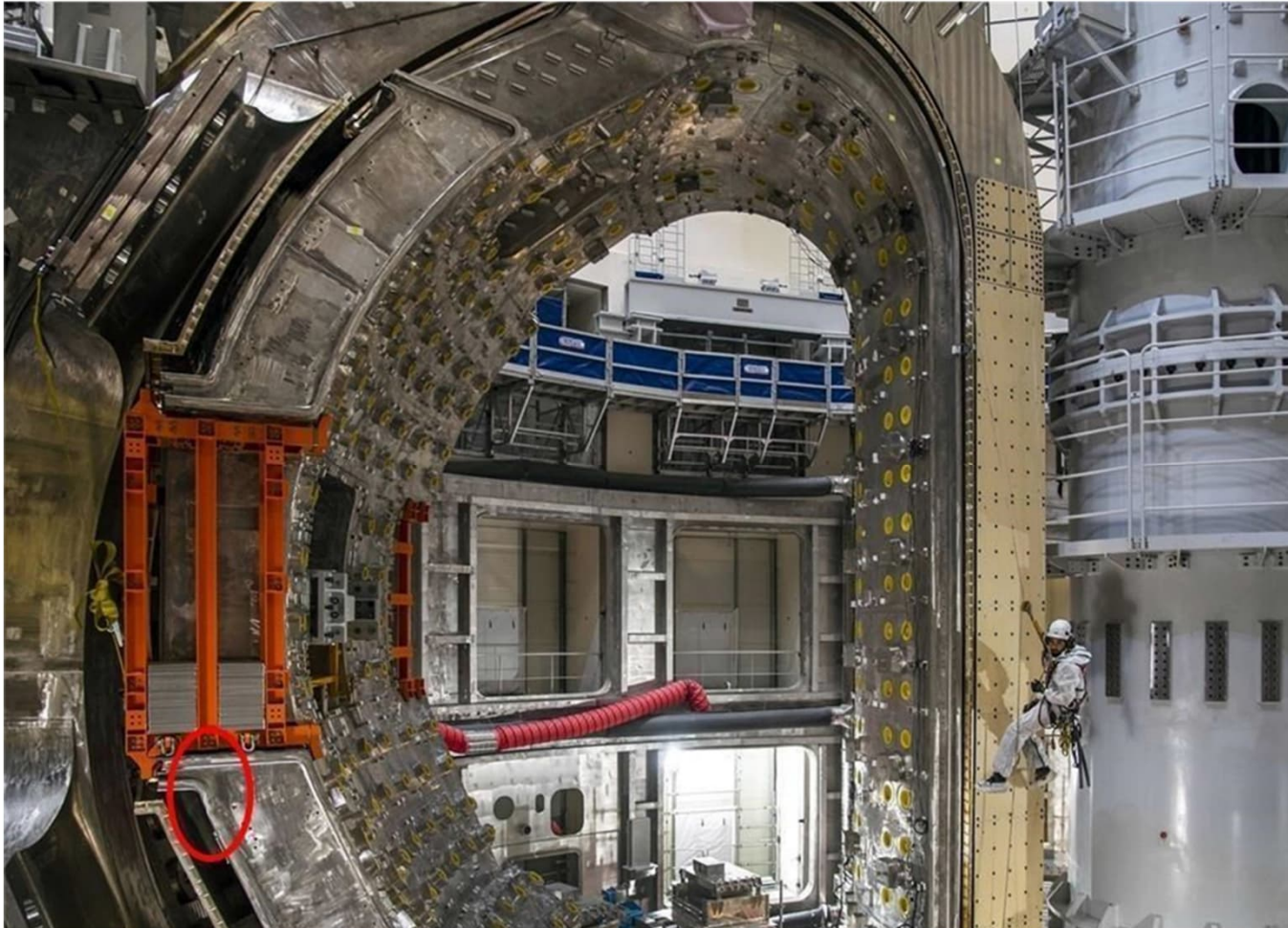


- Stress corrosion cracks (SCC) on thermal shields: problem found on several VV shields

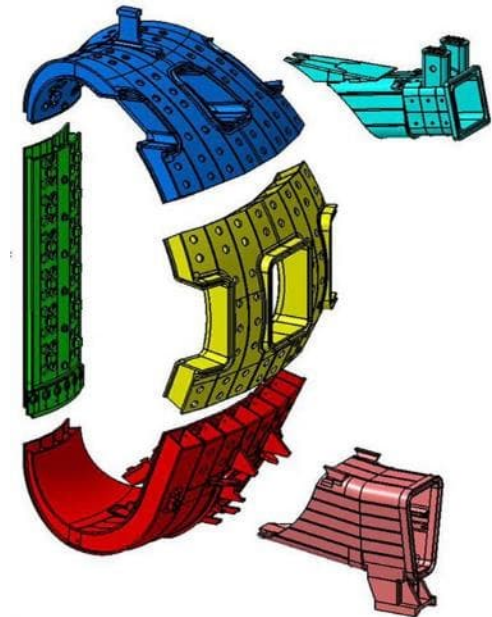




Status of ITER – dimensional non-conformities



Some areas of non-conformity on VV field joints, particularly where the individual VV segments are joined





Status of ITER – Challenges

Dimensional non-conformities in VV field joint contours on the 3 sectors delivered to IO have turned out to be too large to permit sector-sector welding according to required standards.

Stress corrosion cracking (SCC) found on many of the thermal shield cooling pipes.

More details of these issues and their solution can be found in:

<https://www.iter.org/newsline/-/3818>

and

<https://www.iter.org/newsline/-/3830>



Status of ITER – Solutions

For the VV thermal shield:

- remove old pipes and re-weld new ones (different steel and welding process/material)
- re-manufacture of a few panels

For the cryostat thermal shield:

- leave old pipes (but don't connect) and re-weld new ones in-situ (different steel and welding process/material)

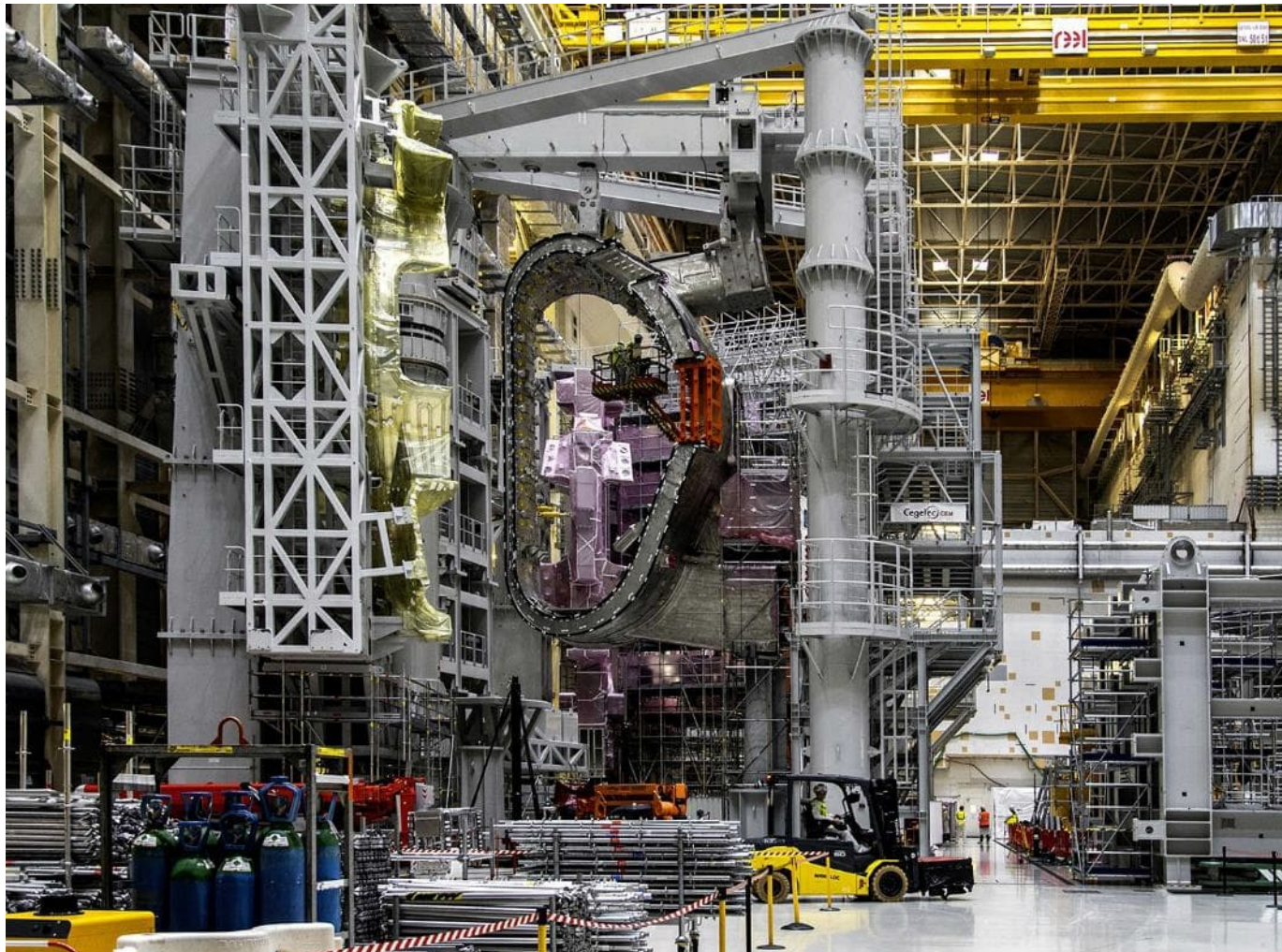
For VV non-conformity:

- remove and add material to meet required dimensions (~73 – 400 kg per sector)

Repairs solidly underway



Status of ITER – Repairs



16 September 2024

- Bevels of VV sector #7 have been restored to nominal geometry
- VV thermal shields repaired and ready to be fitted
- VV sector #7 ready for assembly operations

<https://www.iter.org/newsline/-/4070>



Status of ITER – Repairs



16 September 2024

- Bevels of VV sector #7 have been restored to nominal geometry
- VV thermal shields repaired and ready to be fitted
- VV sector #7 ready for assembly operations

Repairs of all three VV sectors on site at ITER to be completed by July 2025

<https://www.iter.org/newsline/-/4070>



Status of ITER – Research plan and re-baselining

New baseline rationale

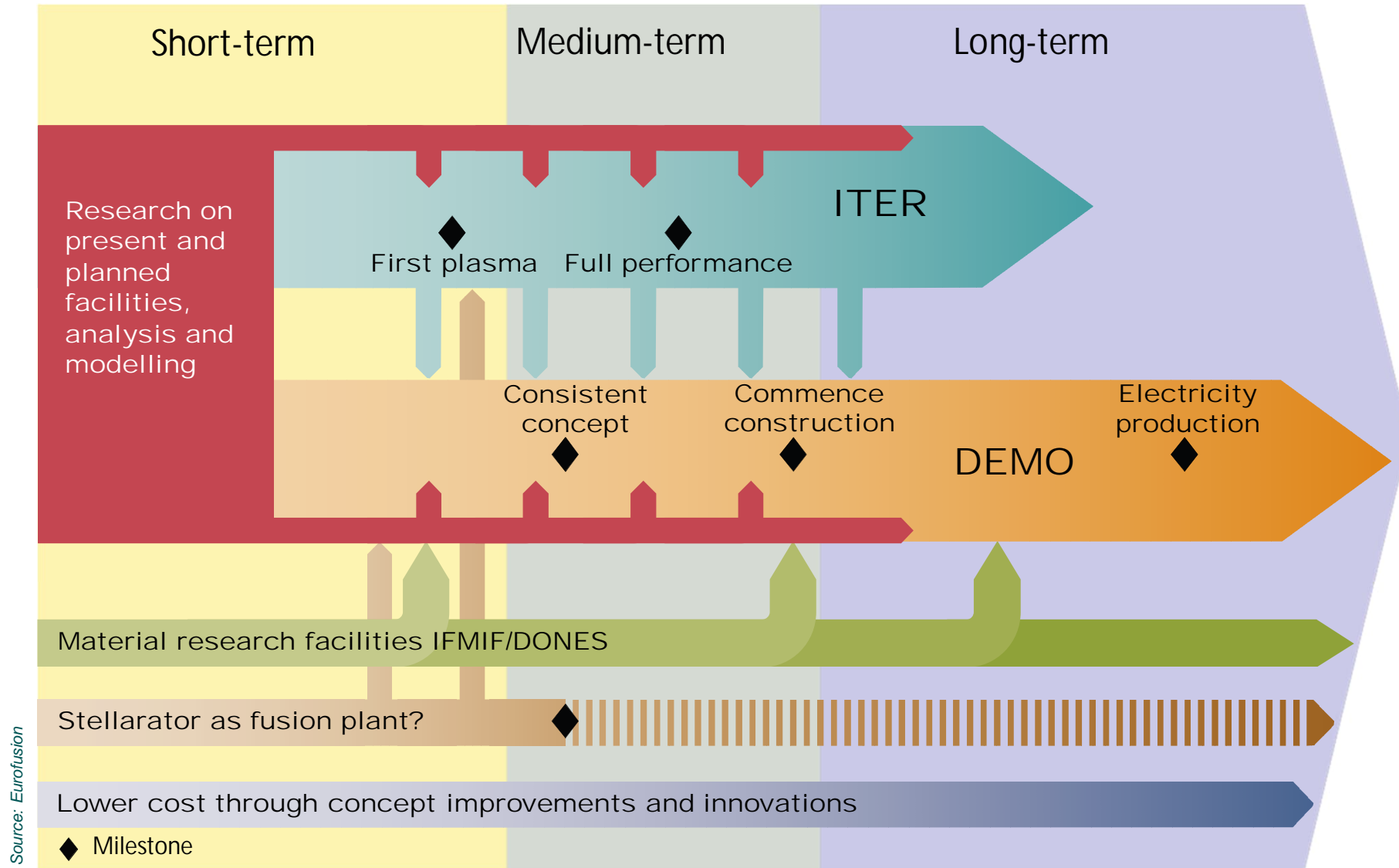
- **Start of Research Operation (SRO) now in 2034** (instead of 2025)
 - Robust achievement of ITER project goals while considering past and current challenges (Delays due to Covid-19 pandemic, Technical challenges of first-of a kind components, Nuclear licensing)
 - Realistic and reliable assembly – commissioning – operation – more complete machine
- **Achievement of earliest start of the ITER Nuclear Phase (DD operation) and minimization of technical risks**
- **Full magnetic energy in 2036** (instead of 2033)
- **Start of DT-operation in 2039** (instead of 2035)
- **Stepwise Safety Demonstration**
 - Gradual increase of neutron fluence
 - Use experimental results from first nuclear phase for safety demonstration of second one

Update research plan in view of new scientific results

- **Implement first wall fully out of W**
- **Optimized heating mix and conditioning of first wall → ease path to $Q = 10$ with added W**



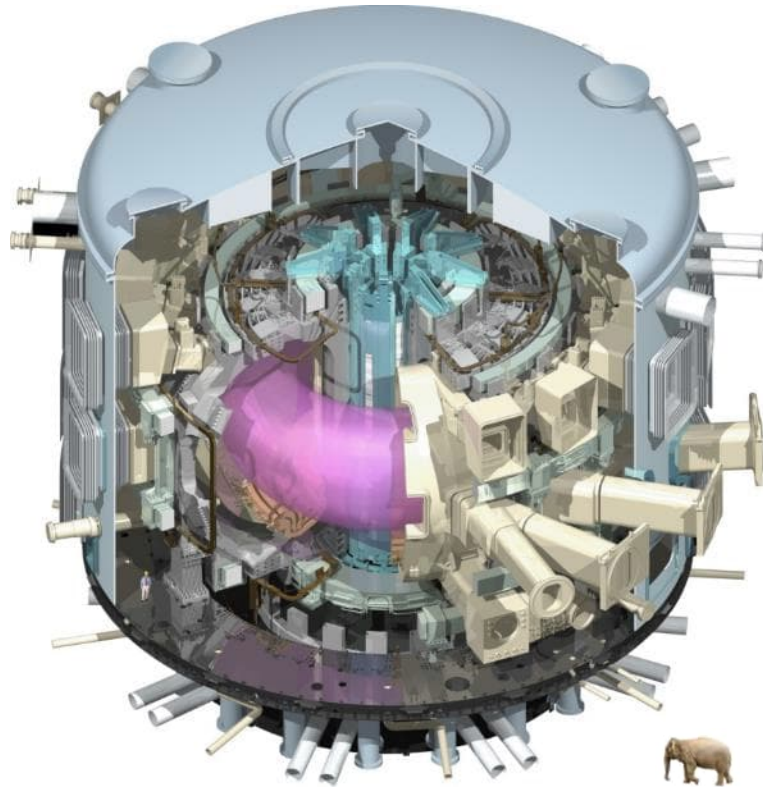
EU Roadmap to Fusion Power



Fusion Power Plants

From experimental device to a power plant

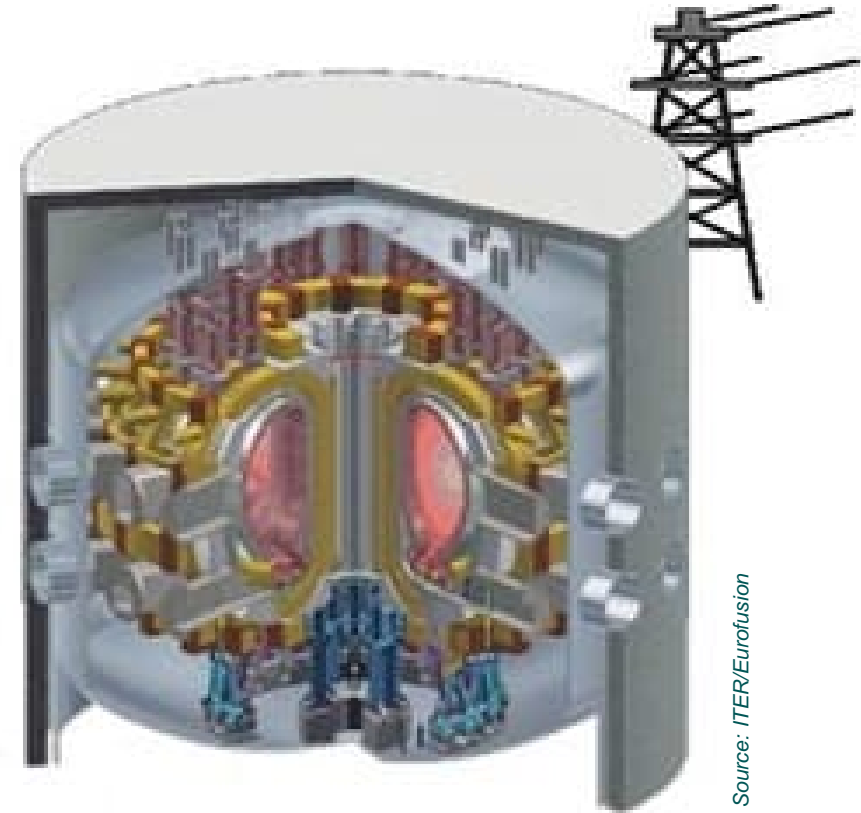
ITER



Plasma pulses ≤ 30 min.
Tritium added
Experimental device and operation



DEMO



Continuous operation
Tritium breeding
Connection to grid



A size comparison





The worldwide fusion landscape – IAEA’s Fusion Device Information System (FusDIS)

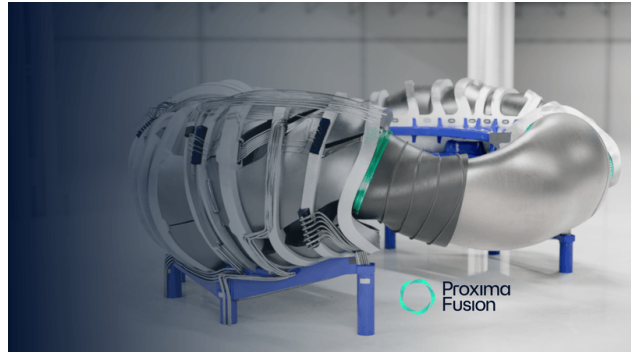
<https://nucleus.iaea.org/sites/fusionportal/Pages/FusDIS.aspx>





From the worldwide fusion landscape – Start-ups in Germany

Proxima Fusion



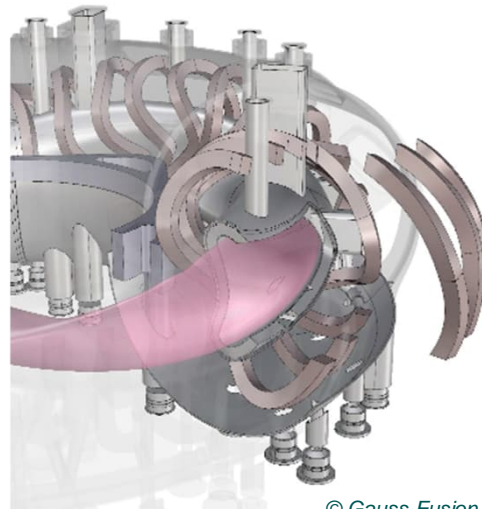
© Proxima Fusion

- Founded by IPP PostDocs
- Aims for Quasi-isodynamic stellarator DEMO

Gauss Fusion



- Founded by European industry involved in ITER construction
- Aims at 1GW DEMO plant based on stellarator



© Gauss Fusion

Focused Energy



- Founded 2021 as German GmbH and US Inc.
- Modular path to laser-driven inertial fusion
- Starts with lab for mass production of targets near Darmstadt

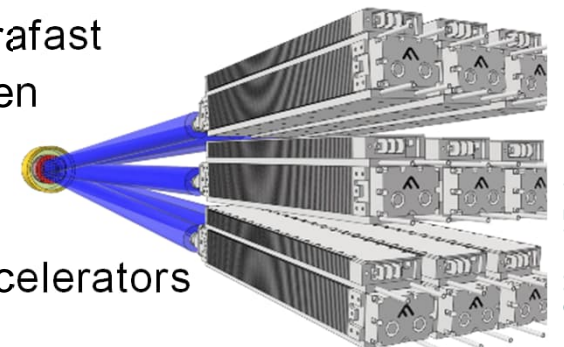


© Focused Energy

Marvel Fusion



- Founded 2019 near Munich
- Aims at direct drive, ultrafast high intensity laser driven fusion
- Development of lasers and nano-structured accelerators



© Marvel Fusion

From the worldwide fusion landscape – STEP (UK)

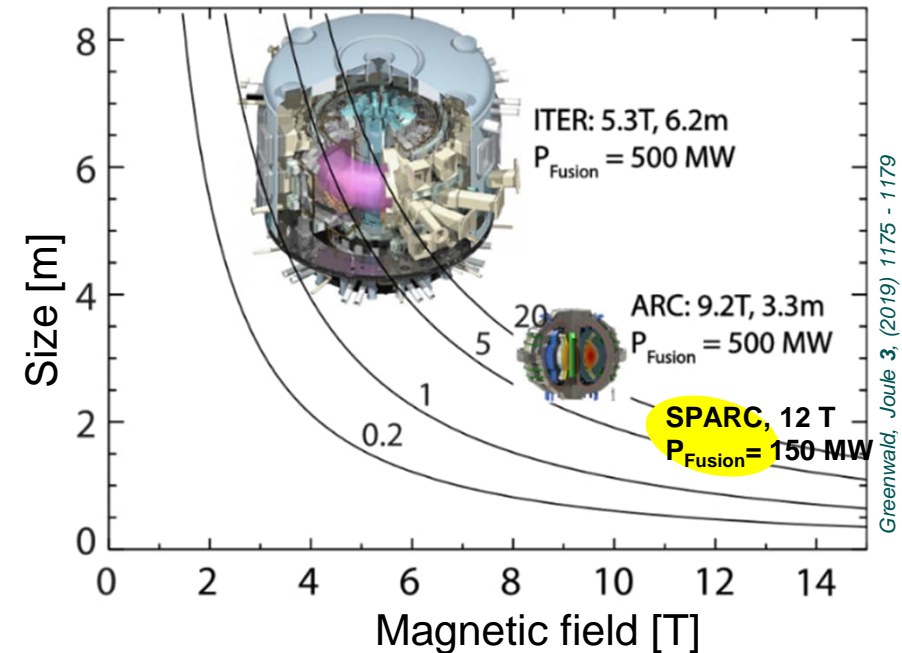
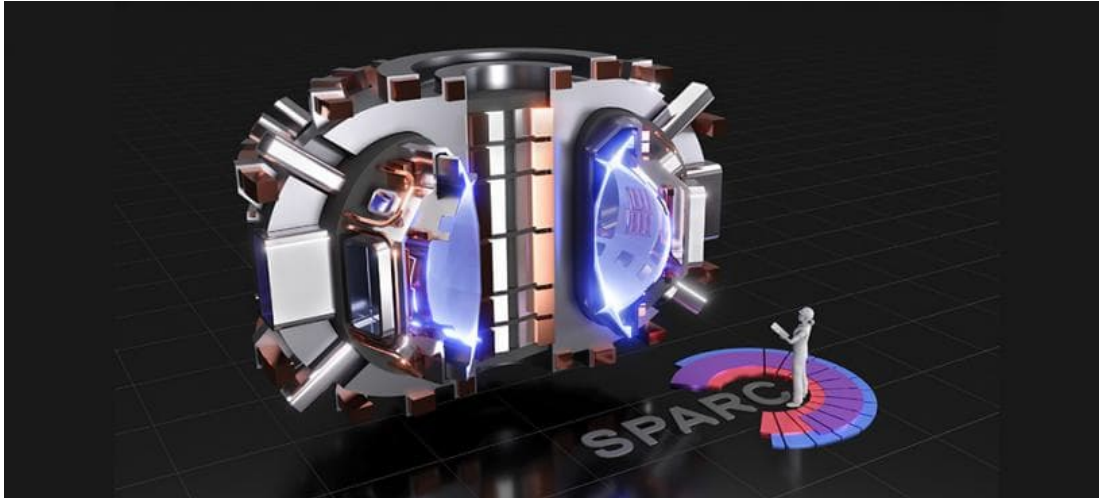
- **STEP Spherical Tokamak for Energy Production**
 - DEMO-class successor device to ITER
 - Proof-of-concept of a fusion plant
 - Scheduled to achieve a 'burning plasma' in 2035
 - Aims to produce net electricity from fusion ~2040
 - Funded by UKAEA
 - Site chosen
 - Conceptual design ongoing
 - UK actively pursues a licensing process dedicated to fusion



Copyright: UKAEA



From the worldwide fusion landscape – SPARC (USA)

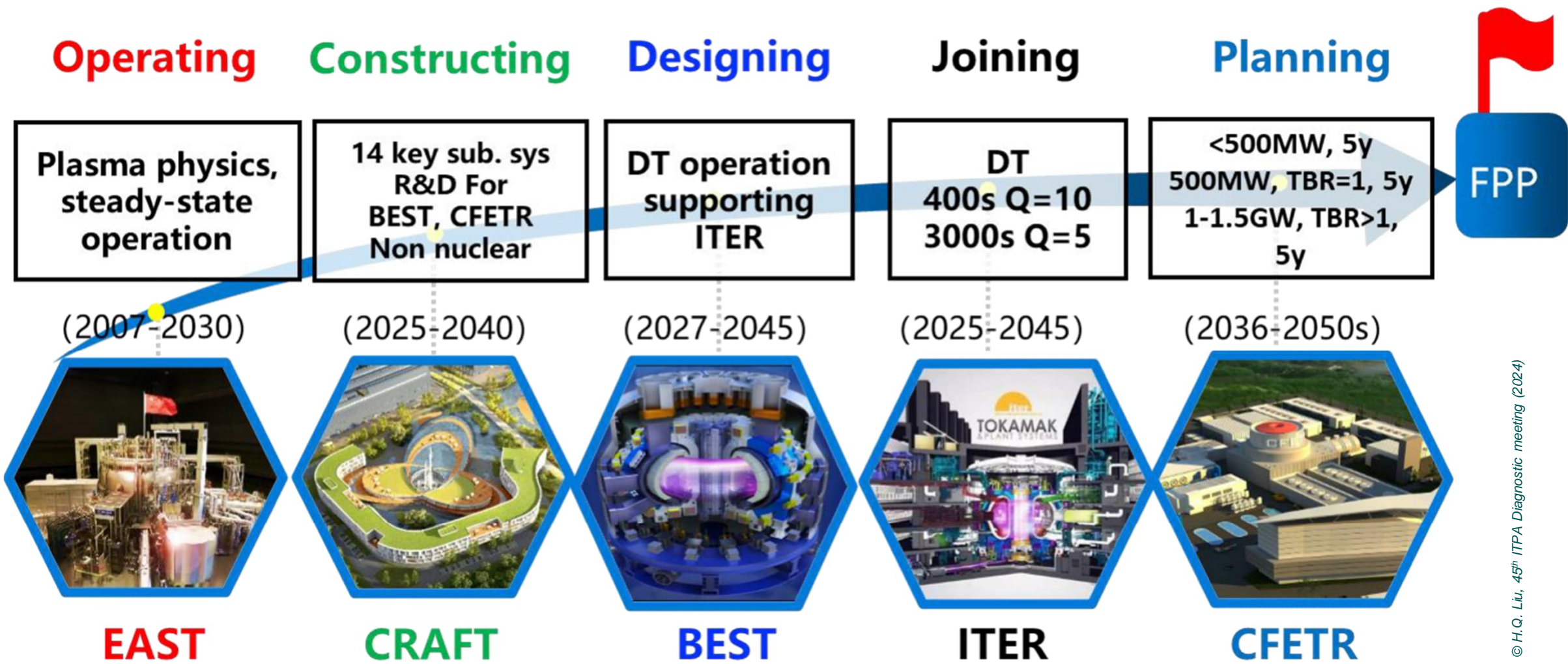


Commonwealth Fusion System (Devens, MA, USA)

- Private company, raised >2 B\$
- Advanced Tokamak with $B_t \sim 12$ T, high-temperature superconductors
- Start of operation in 2027, only short pulses
- Successor ARC aims with half size of ITER and $B_t = 9.2$ T at 400 MW net power



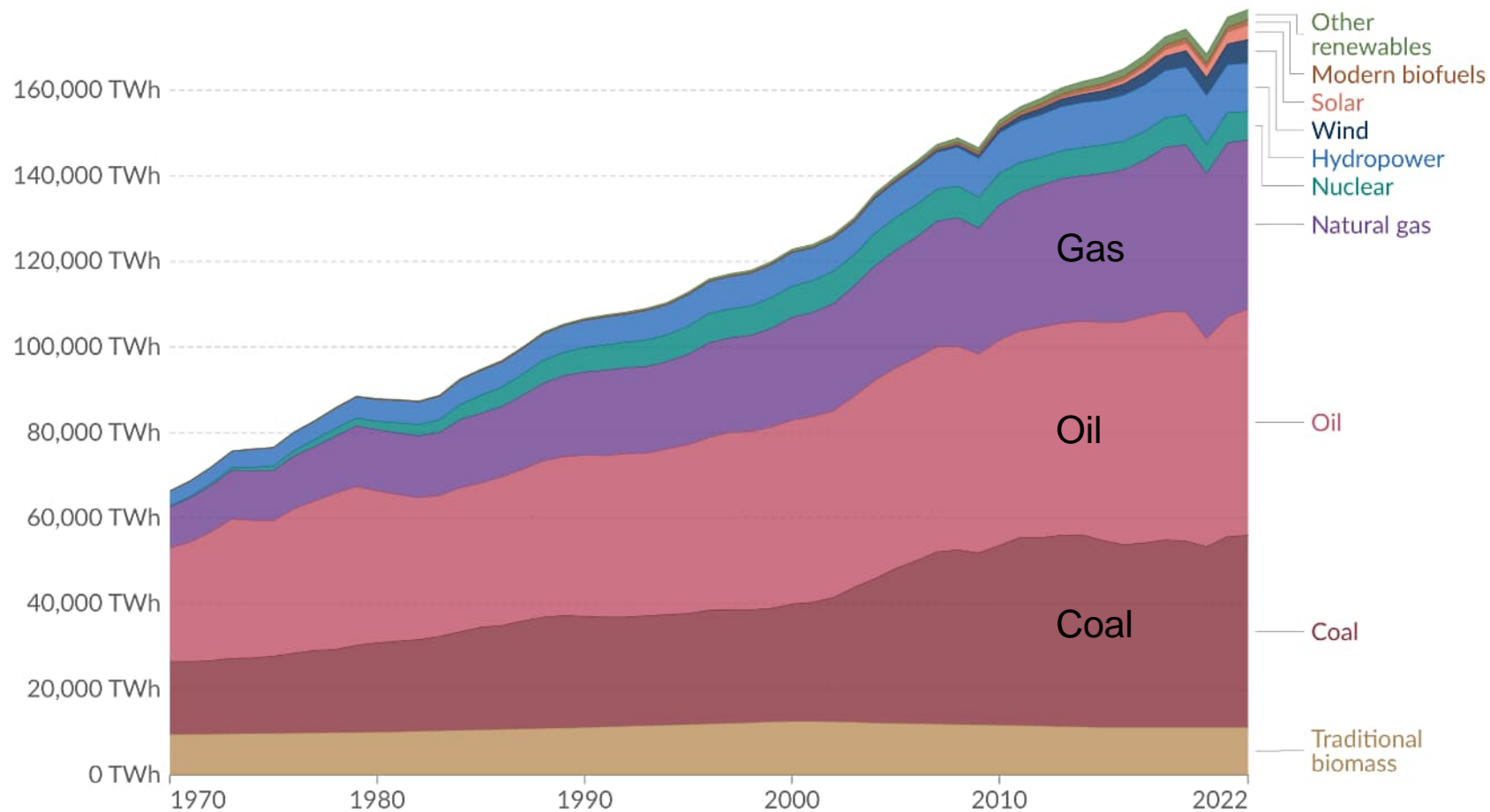
From the worldwide fusion landscape – China's roadmap



© H.Q. Liu, 45th ITPA Diagnostic meeting (2024)



Do we still need nuclear fusion?



Source: Energy Institute - Statistical Review of World Energy (2023); Smil (2017) [OurWorldInData.org/energy](https://www.ourworldindata.org/energy) |CC BY
Note: In the absence of more recent data, traditional biomass is assumed constant since 2015. Primary energy is based on the substitution method.

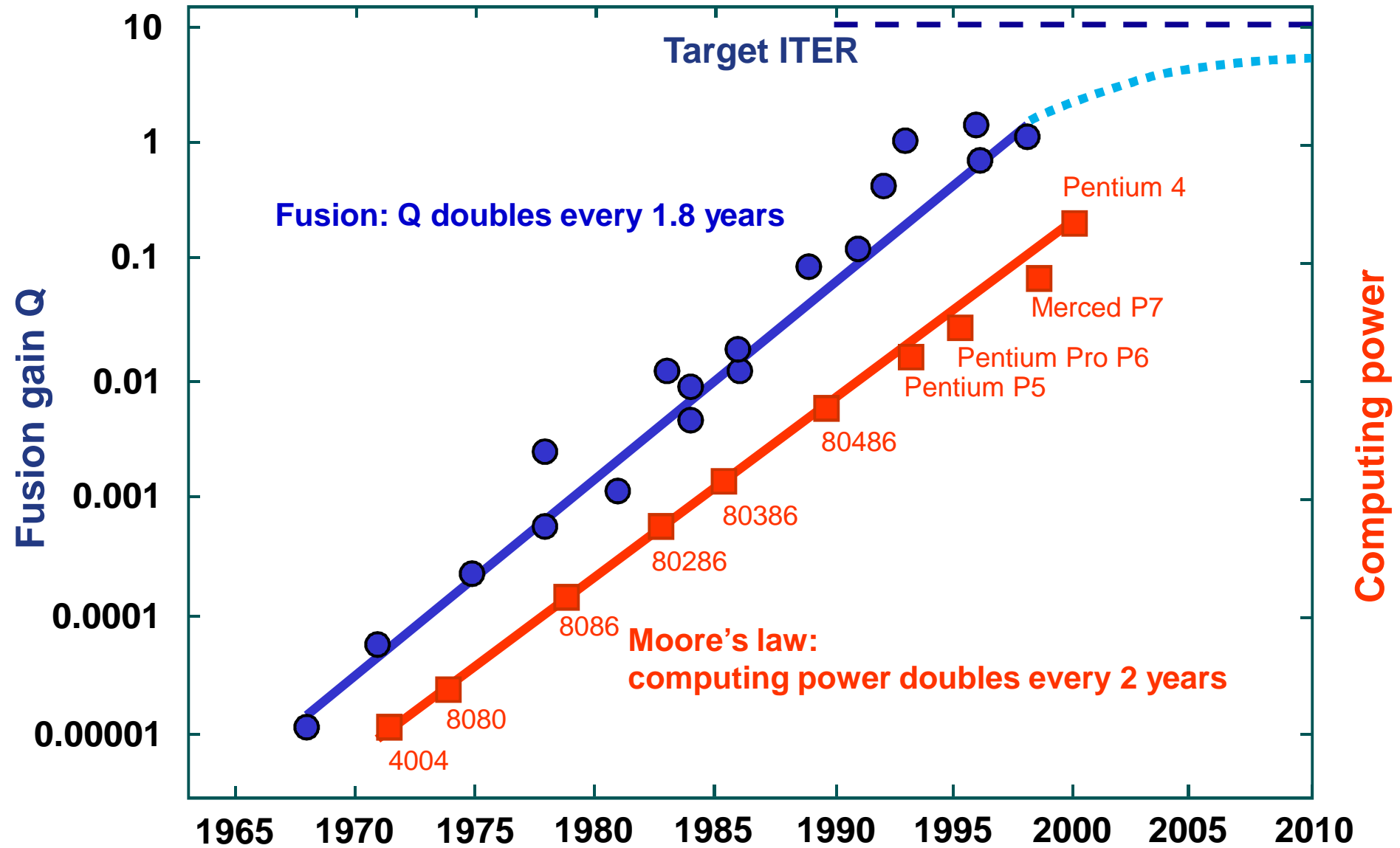
Development of primary energy consumption worldwide

- Need for energy continues to increase
- More than 75% of primary energy production produces CO₂ emissions

Nuclear fusion would be a climate friendly alternative



Progress of fusion





Perspectives for fusion as an energy source

Initial steady progress could not be kept

- Increased size of devices required more funding than provided
- Anticipated budgets have not been approved
- Political agreements were more challenging than anticipated

Physics and technological challenges

- A number of challenges still to be overcome
- Recent progress gives confidence in achieving $Q > 1$

New dynamics in fusion landscape

- > 45 start-ups embarked in fusion
- > 8 B€ private funds invested
- Interest of industry awakens, parallel developments

From ITER's history

- 1985: Gorbachev's proposal to Reagan: international fusion experiment
- 1988: Start conceptual design
- 1992: „Engineering Design Activities“
- 1998: First design rejected (too expensive)
USA leave
Re-design started
- 2001: Approval of design
- 2003: China and South-Korea join
USA returns
- 2005: Agreement on site
⇒ Cadarache (F)
India joins
- 2006: ITER Agreement signed
- 2007: „Broader Approach“ Agreement
- 2008: Start of construction



Conclusions

Fusion as energy source is extremely interesting but also challenging

ITER

- **World's largest first of a kind fusion device**
- **Works to overcome political, social, organizational and technical challenges**
- **Despite delays still relevant and necessary – only project that will tackle all challenges to scale**

Recent successes strongly increased public interest and financial options opening new dynamics in fusion landscape



And how long will it take now for fusion to become reality?

The fusion joke:

„Fusion is always 30-50 years away from realisation.“

Fusion start-ups claim:

„We will erect a power plant within 10-20 years.“

A researcher's opinion:

„20 billion €, 20 years“

If funding for fusion is substantially increased and legal frameworks properly defined,
a prototype power plant is feasible within two decades.



Thanks for your attention!